

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT Fort Calhoun Station
DATE Oct. 4, 1985
COMPLETED BY T. P. Matthews
TELEPHONE (402)536-4733

MONTH September, 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>469.6</u>	17	<u>476.6</u>
2	<u>469.3</u>	18	<u>475.3</u>
3	<u>468.1</u>	19	<u>472.5</u>
4	<u>468.5</u>	20	<u>472.3</u>
5	<u>466.2</u>	21	<u>474.3</u>
6	<u>465.3</u>	22	<u>478.4</u>
7	<u>463.9</u>	23	<u>480.0</u>
8	<u>462.6</u>	24	<u>480.8</u>
9	<u>465.0</u>	25	<u>481.7</u>
10	<u>469.1</u>	26	<u>481.8</u>
11	<u>473.8</u>	27	<u>478.6</u>
12	<u>474.9</u>	28	<u>97.4</u>
13	<u>476.3</u>	29	<u>0.0</u>
14	<u>477.5</u>	30	<u>0.0</u>
15	<u>478.5</u>	31	<u></u>
16	<u>477.4</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

8510230313 850930
PDR ADOCK 05000285
R PDR

JE24
1/1

OPERATING DATA REPORT

DOCKET NO. 50-285
 DATE October 4, 1985
 COMPLETED BY T.P. Matthews
 TELEPHONE (402) 536-4733

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: September, 1985
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 502
7. Maximum Dependable Capacity (Net MWe): 478

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>720.0</u>	<u>6,551.0</u>	<u>105,337.0</u>
12. Number Of Hours Reactor Was Critical	<u>661.5</u>	<u>6,466.1</u>	<u>81,746.3</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>1309.5</u>
14. Hours Generator On-Line	<u>658.8</u>	<u>6,455.5</u>	<u>81,122.9</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>974,986.7</u>	<u>9,564,277.6</u>	<u>103,751,044.6</u>
17. Gross Electrical Energy Generated (MWH)	<u>324,508.0</u>	<u>3,214,944.0</u>	<u>33,984,569.0</u>
18. Net Electrical Energy Generated (MWH)	<u>309,015.2</u>	<u>3,066,256.1</u>	<u>32,477,893.4</u>
19. Unit Service Factor	<u>91.5</u>	<u>98.5</u>	<u>77.0</u>
20. Unit Availability Factor	<u>91.5</u>	<u>98.5</u>	<u>77.0</u>
21. Unit Capacity Factor (Using MDC Net)	<u>89.8</u>	<u>97.9</u>	<u>67.0</u>
22. Unit Capacity Factor (Using DER Net)	<u>89.8</u>	<u>97.9</u>	<u>64.8</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>0.0</u>	<u>3.5</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

The 1985 Refueling Shutdown commenced September 28, 1985, with startup tentatively scheduled for December 11, 1985

25. If Shut Down At End Of Report Period, Estimated Date of Startup: December 11, 1985
26. Units In Test Status (Prior to Commercial Operation): N/A

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

Forecast	Achieved
_____	_____
_____	_____
_____	_____

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH September, 1985

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun Station
 DATE Oct. 4, 1985
 COMPLETED BY T. P. Matthews
 TELEPHONE (402) 536-4733

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
85-01	850928	S	61.2	C	1	N/A	xx	xxxxx	1985 refueling outage commenced September 28, 1985

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

(9/77)

⁵
 Exhibit I - Same Source

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending September, 1985.

1. Scheduled date for next refueling shutdown. October, 1985
2. Scheduled date for restart following refueling. December, 1985
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes

- a. If answer is yes, what, in general, will these be?

Technical Specification changed submitted
September 3, 1985

- b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.

- c. If no such review has taken place, when is it scheduled?

4. Scheduled date(s) for submitting proposed licensing action and support information. Submitted
September 3, 1985
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Submitted and approved

6. The number of fuel assemblies: a) in the core 133 assemblies
b) in the spent fuel pool 305 "
c) spent fuel pool storage capacity 729 "
d) planned spent fuel pool storage capacity May be increased via fuel pin consolidation "
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1996

Prepared by

J. K. Gayer

Date October 1, 1985

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

September, 1985
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at 100% power until September 27, 1985, when a power reduction was started in preparation for the 1985 refueling outage. On September 28, after 195 days of continuous operation, the unit was taken off line. Turbine overspeed and other equipment tests were performed. Prior to commencing plant cooldown, hot degassing of the reactor coolant system was performed for the first time with very favorable results.

Three Auxiliary Operators-Nuclear were hired during September. The following training classes have been in progress through the month: SRO upgrade with four candidates, Equipment Operator-Nuclear (Turbine) with seven students, Auxiliary Operator-Nuclear with four students, operator requalification, C/RP technician, emergency plan and general employee training.

An Unusual Event was declared on September 8, 1985, due to a leaky chlorine bottle which was quickly contained. Poison pins for use in the Cycle 10 core have been loaded in the new fuel.

No safety valve or PORV challenges or failures occurred.

A. PERFORMANCE CHARACTERISTICS

None

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

None

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure

Description

SP-FAUD-1

Fuel Assembly Uplift Condition Detection.

This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved the evaluation of data from a surveillance test to verify that a fuel assembly uplift condition did not exist.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL
(Continued)

<u>Procedure</u>	<u>Description</u>
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SP-SGLVT-1	Steam Generator Level Test.
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This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since the steam generator level was merely lowered to near its design level 64% (temporarily) in an effort to document steam quality changes. Upon completion of the special procedure, the steam generator operating levels were returned to normal operating levels (i.e., approximately 70%).

System Acceptance Committee Packages for September, 1985:

<u>Package</u>	<u>Description/Analysis</u>
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EEAR FC-70-11	New Fuel Elevator/Spent Fuel Interlock.
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This modification adds limit contacts required to prevent accidental insertion of a spent fuel assembly into the new fuel elevator and raising it out of the water. This modification does not have an adverse effect on the safety analysis.

EEAR FC-82-72	Chemical Lagoon Influent pH.
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This modification was installed to ensure effluent from the water plant to the waste lagoons is with a specified pH range. This pH will be controlled via a "holding basin" which has accessories to raise or lower basin pH as necessary. Therefore, all effluent to the lagoons are of proper pH. This modification does not have an adverse effect on the safety analysis.

EEAR FC-83-107	Intake Structure Ventilation.
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This modification provides forced air cooling to the circulating water pumps' upper bearings. This modification does not have an adverse effect on the safety analysis.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL
(Continued)

System Acceptance Committee Packages for September, 1985: (Continued)

<u>Package</u>	<u>Description/Analysis</u>
EEAR FC-84-93	<p>Undersized HGA Relays in Control Circuits for HCV-247, 248, 249 and 2988.</p> <p>This modification eliminated or changed control relays internal to the circuits/circuitry of valves HCV-247, 248, 249 and 2988. The changeout was warranted since, under a specific sequence of operating events, the existing relays would arc. This modification does not have an adverse effect on the safety analysis.</p>
EEAR FC-84-158	<p>E-Field 14 Relocation.</p> <p>This modification expanded the E-field (Zone 14) so that a previously known "dead zone" would be eliminated. This modification does not have an adverse effect on the safety analysis.</p>
EEAR FC-85-04	<p>Alarm Indication for Door 989-20.</p> <p>This modification eliminates or isolates wiring previously shared between Door 989-19 and 989-20. Previously, the alarm for 989-19 would alarm regardless of which door (989-19 or 989-20) was opened. This modification allows for each door to trigger a separate alarm. This modification does not have an adverse effect on the safety analysis.</p>
EEAR FC-85-37	<p>Replacement of Perimeter Security Cameras.</p> <p>This modification changed the perimeter security cameras to a different brand/model. The function of the cameras remained unchanged. This modification does not have an adverse effect on the safety analysis.</p>
EEAR FC-85-81	<p>Security Fence.</p> <p>This modification allowed for alteration of the existing security fence in order to upgrade to existing security plan standards. This modification does not have an adverse effect on the safety analysis.</p>

E. RESULTS OF LEAK RATE TESTS

Refueling leak testing is in progress. Total leakage will be reported after testing is completed.

F. CHANGES IN PLANT OPERATING STAFF

During September, Fred Klauzer, Bill McMillian and John Goodell reported to work as Auxiliary Operators-Nuclear.

G. TRAINING

During September, Emergency Operating Procedure training for licensed operators commenced. Training continued for auxiliary operators, turbine room operators, SRO candidates, chemistry technicians and health physics technicians.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

None

II. MAINTENANCE (Significant Safety Related)

A report of all significant safety related maintenance performed during the 1985 refueling outage will be submitted at the conclusion of the outage.

W. Gary Gates

W. Gary Gates
Manager
Fort Calhoun Station

Omaha Public Power District
1623 Harney Omaha Nebraska 68102
402/536-4000

October 11, 1985
LIC-85-456

Mr. James M. Taylor, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

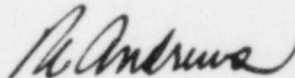
Reference: Docket No. 50-285

Dear Mr. Taylor:

September Monthly Operating Report

Please find enclosed ten (10) copies of the September, 1985 Monthly Operating Report for the Fort Calhoun Station Unit No. 1.

Sincerely,



R. L. Andrews
Division Manager
Nuclear Production

RLA/TPM/rs

Enclosures

cc: NRC Regional Office
Office of Management & Program Analysis (2)
Mr. R. R. Mills - Combustion Engineering
Mr. T. F. Polk - Westinghouse
Nuclear Safety Analysis Center
INPO Records Center
American Nuclear Insurers
NRC File

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11