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NOVEMBER 7-9, 1996

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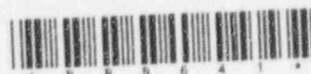
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Station (Memorandum to James M. Taylor, Executive Director for Operations, NRC, from John T. Larkins, Executive Director, ACRS, dated November 18, 1996) - Consistent with the Committee's decision, Dr. Larkins informed Mr. Taylor that the Committee decided not to review the subject reports at this time. The Committee, however, may hear a briefing after the licensee has completed the proposed modifications identified in the subject reports.

APPENDICES

- I. Federal Register Notice
- II. Meeting Schedule and Outline
- III. Attendees
- IV. Future Agenda and Subcommittee Activities
- V. List of Documents Provided to the Committee

MINUTES OF THE FOUR HUNDRED THIRTY-SIXTH MEETING OF THE
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
NOVEMBER 7-9, 1996
ROCKVILLE, MARYLAND

The 436th meeting of the Advisory Committee on Reactor Safeguards was held at Conference Room 2B3, Two White Flint North Building, Rockville, Maryland, on November 7-9, 1996. The purpose of this meeting was to discuss and take appropriate action on the items listed in the attached agenda. A portion of the meeting was closed to discuss information of a personal nature, the release of which would constitute a clearly unwarranted invasion of personal privacy. The rest of the meeting was open to public attendance. There were no written statements nor requests for time to make oral statements from members of the public regarding the meeting.

A transcript of selected portions of the meeting was kept and is available in the NRC Public Document Room at the Gelman Building, 2120 L Street, N.W., Washington, D.C. [Copies of the transcript are available for purchase from Neal R. Gross and Co., Inc., 1323 Rhode Island Avenue, N.W., Washington, D.C. 20005.]

ATTENDEES

ACRS Members: Dr. Thomas S. Kress (Chairman), Dr. Robert L. Seale (Vice-Chairman), Dr. George Apostolakis, Mr. John Barton, Dr. Ivan Catton, Dr. Mario H. Fontana, Dr. Don W. Miller, Dr. Dana A. Powers, and Dr. William J. Shack. [For a list of other attendees, see Appendix III.]

I. CHAIRMAN'S REPORT (Open)

[Note: Dr. John T. Larkins was the Designated Federal Official for this portion of the meeting.]

Dr. Thomas S. Kress, Committee Chairman, convened the meeting at 8:30 a.m. and reviewed the schedule for the meeting. He announced some minor changes to the timing of certain items on the agenda and drew the attention of the Members to the speeches recently given by Chairman Jackson.

II. PROPOSED RULE ON STEAM GENERATOR INTEGRITY (Open)

[Note: Mr. N. Dudley was the Designated Federal Official for this portion of the meeting.]

Introduction

Dr. Robert Seale, Acting Chairman of the Joint Materials & Metallurgy and Severe Accidents Subcommittee, introduced the session by noting the scope of the proposed risk-informed,

performance-based rulemaking and identified the questions that were expected to be answered by the staff and industry. Dr. Seale introduced Dr. Joam Hopenfeld, who had filed a differing professional opinion regarding the voltage-based alternate repair criteria.

Professional Differing Opinion

Dr. Hopenfeld stated that the assumed induced core melt frequency with containment bypass should be 10^{-4} events per year instead of the 10^{-6} events per year used by the staff. He stated that during severe accidents steam generator tubes with flaws will fail before the pressurizer surge line. He stated that uncertainties associated with characterizing steam generator tube defects and severe accident phenomena are not sufficiently understood to properly model tube rupture events. His examples included crack networking of flaws not being adequately determined by nondestructive examinations, increased heat transfer caused by flow through multiple tube cracks not being included in thermal hydraulic calculations, and steam jets from tube cracks not being evaluated as a possible failure mechanism for adjacent tubes.

Dr. Hopenfeld and the Committee Members discussed the technical analyses that supported his conclusions and technical details related to his examples. Dr. Hopenfeld concluded that the staff should document the assumptions and models used to study hidden uncertainties.

NRC Staff Presentation

Mr. Jack Strosnider, NRR, presented the objectives of and schedule for the proposed rule and regulatory guide related to steam generator tube integrity. He explained the policy issues related to the flexibility provided by the performance-based approach and the use of severe accident risk within the regulatory framework. Mr. Strosnider summarized the attributes and detailed the requirements of the proposed rule. He explained the deterministic and probabilistic structural integrity performance criteria, and the operational and accident leakage performance criteria. Mr. Strosnider and the Committee Members discussed whether licensee methodologies for determining compliance with the performance criteria should be approved by the staff. They also discussed the risk worth of the proposed rule, calculating fission product deposition in the secondary system, and the effect of human error on calculating core damage frequency caused by steam generator tube ruptures.

Mr. Joseph Donoghue, NRR, presented the basis for the rule and noted that the fundamental objective of the rule is to maintain defense-in-depth with respect to the containment function of the

tubes. Mr. Donoghue explained that the results of an example analysis indicate a containment bypass frequency above the subsidiary safety goal. He summarized the following different components of the example analysis:

- representative flaw distribution,
- event tree quantification,
- reactor coolant pressure boundary weak points,
- thermal-hydraulic results, and
- tube performance model.

Mr. Donoghue identified the key issues and limitations related to the different components. He explained that if a licensee could not demonstrate that a plant met the initiating event criterion or the conditional tube failure probability criterion, then plant risk would need to be reduced by a combination of plant and procedural modifications. The staff and the Committee Members discussed the effect of enhanced heat transfer due to increased tube leakage, and the possible use of risk limits instead of frequency limits as performance criteria.

Industry Presentation

Mr. Richard Pearson, Northern States Power Company, explained that the industry would like a relaxation in the repair limits related to steam generator tubing, but noted that industry considers the proposed regulatory requirements to be onerous. The industry, nevertheless, is cooperating with the staff in an effort to reduce the number of details contained in the proposed rule and regulatory guide. The industry would like the staff to define requirements and let the industry develop procedures for implementing them. The industry would like to address severe accidents in severe accident space and not as part of a regulatory requirement. Mr. Pearson and the Committee Members discussed leaving cracked tubes in service in order to measure crack growth rates, and the staff review of licensee-developed methodologies and alternate repair criteria correlations.

CONCLUSION

The Committee issued a letter to the Executive Director for Operations, dated November 20, 1996, on this matter.

III. RISK-BASED ANALYSIS OF REACTOR OPERATING EXPERIENCE

[Note: Mr. M. Markley was the Designated Federal Official for this portion of the meeting.]

Dr. Robert Seale, Acting Chairman of the Plant Operations Subcommittee, introduced the topic to the Committee. He noted that joint

meetings of the Subcommittees on Probabilistic Risk Assessment (PRA) and Plant Operations were held on July 17 and October 30, 1996, regarding these matters. He summarized the deliberations of the Subcommittees and introduced Mr. Patrick Baranowsky, Chief, Reliability and Risk Assessment Branch (RRAB), Office for Analysis and Evaluation of Operational Data (AEOD).

AEOD Presentation

Mr. Patrick Baranowsky led the discussion for the NRC staff. Messrs. Edward Jordan, Director, AEOD, and Steven Mays, Chief, Reactor Risk Assessment Section, RRAB, AEOD, provided supporting discussion. Mr. Baranowsky gave an overview of the AEOD/RRAB mission and programs, including a discussion of system reliability studies, risk-based performance indicators (PIs), accident sequence precursor (ASP) studies, and common-cause failures (CCFs). He summarized the background materials provided to the ACRS for review, AEOD's role in the NRC PRA Implementation Plan, planned program accomplishments, and expectation for future meetings with the ACRS. He stated that AEOD was requesting feedback from the Committee regarding the overall approach and direction of these programs. Significant points made during the presentation include:

- The plan for risk-based analysis of reactor operating experience is based on the practical ability to collect and analyze data, accepted state-of-the-art methods to perform analyses, and rational decomposition of risk.
- Current PIs include: automatic scrams while critical, safety system actuations and failures, significant events, the forced outage rate, the equipment forced outage rate per 1,000 critical hours, collective radiation exposure, and cause codes.
- The staff has contracted with the accounting firm of Arthur Andersen, Inc. to perform an independent analysis of the NRC Senior Management Meeting (SMM) process to evaluate the development of indicators that can provide a basis for judging whether a plant should be placed on or deleted from the "watchlist" of problem facilities. The staff noted that the Commission also requested that this study consider methods to make the SMM process more scrutable.
- Candidates for risk-based PIs include: initiating event frequency, risk-important system reliability, ASP-type analysis of significant events, and integrated core damage frequency (CDF).
- Objectives of the ASP program include: identifying and ranking risk significance of operational events, determining generic

implications of operational events, providing supplemental information on plant-specific performance, providing event comparisons with PRAs, and providing empirical indications of industry risk and associated trends.

- The purpose of the system reliability studies is to evaluate reliability and provide engineering insights for risk-important systems based on operating experience. Objectives include: using actual demands, failures, and unavailabilities to estimate reliability; analyzing trends in reliability; quantifying uncertainties; comparing findings with published probabilistic risk assessment/ individual plant examination (PRA/IPE) values; identifying plant-specific differences; and providing engineering insights.
- The CCF database includes characterization of systems, component types, and failure modes. CCF records were screened and classified per NUREG/CR-4780, "Procedures for Treating Common-Cause Failures in Safety and Reliability Studies," and were analyzed using alpha factor and Greek Letter methods.

Dr. Seale questioned the difference between risk-based and risk-informed PIs. The staff stated that the initiative for risk-based PIs was in place before the NRC approach became risk-informed. Mr. Barton questioned how "on-line maintenance" was addressed. The staff stated that PIs were based on operating events and that there was not a specific indicator for on-line maintenance. Drs. Kress and Powers offered the view that core damage frequency (CDF) or an integrated CDF indicator might make a good PI.

Mr. Barton questioned whether the maintenance backlog might serve as a good indicator. The staff stated that all licensees do not count backlog in the same way and added that it does not include consideration of risk. The staff also stated that response time is important for any indicator and that they hope to get some additional insights from the study being completed by Arthur Andersen, Inc. The Committee requested a briefing on the results of this study when available.

Mr. Barton questioned how the staff handles a licensee event report (LER) that may not have been done well. The staff stated that their system reliability studies evaluate LERs to examine actual demands and failures and compares those results to the experience estimated in PRA/IPEs. They also stated that the ASP program looks at determining quantitative representations of risk but noted that sensitivity studies are used for events that are not modeled well or are not modeled at all.

Drs. Seale and Powers questioned the evolutionary nature of the CCF database and associated methodology. In particular, they ques-

tioned whether a more progressive step forward could be made rather than modifying the existing program. They also questioned the proprietary nature of the information provided by the Institute for Nuclear Power Operations (INPO) via the Nuclear Power Reliability Data System (NPRDS). The staff stated that the CCF database has evolved with incremental gains through access to NPRDS and added that there is a legal question with regard to the availability of that data for other purposes. The staff also noted that they were discussing enhanced data sharing with INPO in lieu of a possible reliability data rule. Several ACRS Members expressed the view that any information used by the NRC for determining generic CCFs should also be available publicly and that this information could be used in plant-specific applications.

At the conclusion of the meeting, the staff requested an ACRS letter commenting on AEOD's programs. Dr. Seale summarized his understanding of the context of ACRS comments and concerns. He stated that he believed the overall approach and direction was appropriate. He also emphasized that any set of indicators should have a risk focus and that the development of the CCF database represented a positive statement with regard to the benefits of NRC and industry data sharing.

Conclusion

The Committee issued a letter the Executive Director for Operations dated November 22, 1996, on these matters.

IV. REVISED SOURCE TERM FOR OPERATING REACTORS

[Note: Mr. A. Singh was the Designated Federal Official for this portion of the meeting.]

Introduction

The Chairman of the Severe Accidents Subcommittee, Dr. Fontana, provided the Committee with the following summary of the presentations made by representatives of the NRC staff, Nuclear Energy Institute (NEI), and Entergy Operations, Inc., during the Subcommittee meeting on November 6, 1996. The purpose of the Subcommittee meeting was to gather information on the proposed Commission paper concerning the use of NUREG-1465, "Accident Source Terms for Light-Water Nuclear Power Plants," and the approval of license applications using revised source terms at operating reactors.

Staff Presentation

During the 413th ACRS meeting, September 8-10, 1996, the Committee discussed the proposed final version of NUREG-1465 and issued a

report to NRC Chairman Selin. The report "urged that the risk implications be evaluated and consideration be given to allowing current licensees the option of using the timing assumptions in a proposed source term without performing a complete source term reanalysis." The report also stated that the Committee believed that using realistic source terms could reduce operational plant risk.

On November 15, 1995, NEI submitted EPRI report TR-105909, "Generic Framework for Application of Revised Accident Source Term for Operating Plants" to the NRC for review. Four licensees have submitted licensing amendment requests, which the staff is reviewing as part of a pilot program.

In a Staff Requirements Memorandum (SRM) of July 2, 1996, the staff was requested to return to the Commission for guidance as it proceeded, and not to proceed down a path of de-facto exemptions if rule changes were necessary. In response to the SRM, the staff prepared a proposed Commission Paper that responded to the NEI Generic Framework Document. The NRC stated that the proposed plant changes were classified into four groups: allowable leak rate changes, isolation valve timing changes, filtration unit simplification, and mitigation system actuation timing.

The staff focused its review on the following four fundamental principles described in the NEI Generic Framework Document:

- Continued use of the existing licensing basis is acceptable for operating plants.
- The complete implementation of a new source term as a substitute for the existing licensing-basis is acceptable.
- The selective implementation of the revised source term is acceptable for timing applications only.
- The dose calculations using the limits in 10 CFR Part 100 and current licensing-basis methods are acceptable for revised source term applications.
- The staff determined that a mandatory backfit of the revised source term at operating plants is not required.
- The staff believes that the integrated impact on a plant as a result of implementing the revised source term needs to be assessed.

The views of the staff and the industry have converged since the early draft of the proposed Commission paper. The staff now agrees that rulemaking may not be necessary.

Ms. Sheri Mahoney, Entergy Operations, Inc., commented on the proposed Commission paper and requested that the ACRS endorse early implementation of the revised source term through 10 CFR 50.59.

Mr. Kurt Cozens, NEI, presented NEI's position on implementing the revised source term. The NEI approach would be to perform parallel calculations for Part 100 using a new source term, but following Part 100 requirements.

Conclusion

This Subcommittee Chairman's Summary was for information only.

V. EMERGENCY PLANNING FOR ADVANCED REACTORS

[Note: Dr. M. El-Zeftawy was the Designated Federal Official for this portion of the meeting.]

Dr. Seale stated that the purpose of this session was to hear a briefing by the NRC staff and NEI representatives regarding a simplified approach to emergency planning for advanced reactors.

NRC Staff Presentation

Mr. J. O'Brien, NRR, stated that in an SRM dated July 30, 1993, the Commission requested the staff to "remain open to suggestions to simplify the emergency planning requirements for reactors that are designed with greater safety margins. To that end, the staff should submit to the Commission recommendations for proposed technical criteria and methods to use to justify simplification of existing emergency planning requirements." To respond to the Commission's request, the staff prepared a working paper that summarizes the staff's evaluation of technical criteria and methods, which could be used to justify the simplification of emergency preparedness for reactors with greater safety margins. The staff, however, informed the Commission that because design certification of advanced reactors (PRISM, MHTGR, PIUS, and CANDU) was not being pursued and adequate design and risk assessment information was not available, the staff had changed its focus to concentrate on the passive and evolutionary designs. The staff's recommendations were as follows:

- Emergency planning (EP) should be maintained as an essential element in NRC's defense-in-depth philosophy.
- The rationale for EP requirements for advanced plants should be the same as for current plants and the technical criteria derived from that rationale should be the same; i.e.,

- the emergency planning zone (EPZ) should encompass those areas where the projected dose from design basis accidents could exceed the protective action guidelines (PAGs);
 - the EPZ should encompass those areas where consequences of less severe core melt accidents could exceed EPA/PAGs;
 - the EPZ should be of sufficient size to provide for substantial reduction in early severe health effects in the event of the more severe core melt accidents;
 - detailed planning within the EPZ was expected to provide a substantial base for an ad hoc expansion of response efforts in the event that this was necessary.
- Probability considerations can be used to set reasonable bounds on the planning effort. Simplification of EP requirements for advanced reactors may be warranted due to the increased safety margins for these designs. One potential change that may be appropriate is implementation of a graded emergency response, where the notification requirements for the population beyond a certain distance (e.g. 2 or 5 miles) are relaxed due to the decrease in risk.
 - Elimination of accident types from consideration based upon risk analysis is not recommended for the new designs (e.g. the ABWR) because of the uncertainty associated with such evaluations.
 - Probability considerations are useful in setting reasonable bounds on the planning effort, but are not to be used to define specific bounds to the EPZ size.

Mr. O'Brien responded to some previous ACRS questions, as follows:

- Q. What level of risk is being "accepted" for currently operating LWRs with their existing EPZs? A. The size of the EPZ is not based upon a defined risk level but rather on the potential consequences of an accident if one were to occur. The impact of emergency response actions on estimates of risk was evaluated in NUREG-1150.
- Q. Is this level of "accepted" risk appropriate? If not, what should it be? A. In its Safety Goal policy statement, the Commission established goals that broadly defined an acceptable level of risk of electricity generation via nuclear power, and did not define an acceptable risk level for an

individual plant. Policy issues regarding use of risk information are under consideration.

- Q. For advanced plant designs, what would be the size of the EPZ based on a level of risk comparable to the "accepted" value? What are the implications of this result? A. The emergency planning requirements, including the size of the EPZs, are based not on risk of an accident but rather on the potential consequences of an accident (philosophy of defense-in-depth). In addition, the implications of a low calculated risk of severe accidents for future plants are being considered in developing the new 10 CFR Part 100 siting criteria.

Mr. Ron Simard, NEI, briefed the Committee regarding the industry's views of the basis of emergency response planning appropriate to advanced reactor designs. He stated that emergency planning provides defense-in-depth (i.e., accident prevention, mitigation to minimize releases, and planning to limit the consequences of a release). The EPZ is the area for which prompt and effective action can be taken to protect the public. The PAGs define how much of a potential exposure warrants protective action (one rem for the actual or projected duration of the release).

The bases for determining the size of the EPZ are as follows:

- 10 miles for current LWRs, less for smaller reactors and gas cooled designs. (PAGs would not be exceeded outside the zone for design-basis accidents and for most core melt sequences);
- less than 10 miles for advanced reactors because the size of the potential release (source term) is reduced, the probability of a release is reduced, and the delay time before release is increased; and
- current ALWR designs based on the Utility Requirements Guideline is dose at 0.5 miles less than one Rem for 24 hours.

There are three areas in the emergency planning concept: the Reactor area, the Response area, and the Awareness area. The Response area is that area close to the plant within which there should be provisions for prompt notification and protective actions. The size is determined by dose at the boundary and the capability for rapid response. The response area will have a complete set of planning actions similar to existing emergency response plans. The licensee is responsible for planning, prompt notification, radiological analysis, etc. The awareness area is that area, beyond the Response area, within which the radiological effects (if any) would be small and would take place over a longer time frame. The size and shape of this area would be based on factors other than dose (site characteristics, capability of local

governments to respond to industrial emergencies). The planning addresses actions over a longer time frame (e.g. a day or longer). The offsite agencies are responsible for planning, implementation with support from the licensee (e.g., dose assessment, field monitoring, radiological training).

CONCLUSION

This briefing was for information only.

VI. NITROGEN BUBBLE IN THE REACTOR COOLANT SYSTEM AT THE HADDAM NECK NUCLEAR POWER PLANT

[Note: Mr. P. Boehnert was the Designated Federal Official for this portion of the meeting.]

Subcommittee Chairman's Comments

Mr. John Barton, Chairman of the Plant Operations Subcommittee, introduced this topic to the Committee. He noted that the purpose of this discussion was for the ACRS to be briefed on the results of the NRC Augmented Investigation Team (AIT) investigation of the August 28, 1996 event at the Haddam Neck Nuclear Power Plant. As a result of this event, a large nitrogen bubble formed in the head of the reactor vessel.

In describing the event, Mr. Barton stated that the safety significance of the nitrogen intrusion into the RCS was the potential for loss of decay heat removal via the residual heat removal (RHR) system. Further, the event could have complicated the restoration of decay heat removal via the steam generators as well. He also stated that NRR has drafted an NRC Information Notice to address the generic implications of this event.

Finally, Mr. Barton said the NRC staff would provide the details of the event and discuss its generic implications. He also noted the presence of two representatives of the Haddam Neck licensee and said that they were welcome to comment as they deemed necessary, and could respond to any questions from the Committee.

NRC Presentation

Mr. Jim Trapp, Region I, briefed the Committee on the results of the AIT investigation of the Haddam Neck plant event and related incidents that resulted in challenges to decay heat removal. Key points of his brief included:

- The AIT investigated three major aspects of the event: the undetected accumulation of nitrogen gas in the vessel head,

two inadvertent diversions of reactor coolant system inventory, and, equipment failures in the decay heat removal system.

- Regarding the gas intrusion: due to an inadvertent opening of a valve, nitrogen gas began leaking into the RCS (the valve did not fully reseal upon its closure). The gas leakage continued over the next four days, resulting in the formation of a bubble in the vessel head that depressed the water level about 3.5 feet below the vessel flange. Due to a combination of a lack of available instrumentation for direct measurement of the vessel level (the RVLIS and core thermocouples were disconnected for refueling), and an inadequate backup level indicator (which resulted in only the pressurizer level being available), the operators did not detect the bubble growth in a timely manner. The event was terminated as a result of an investigation to determine the source of excessive nitrogen use.
- Regarding the other two major aspects investigated: (1) as a result of plant operator errors, water (500 and 300 gallons) was diverted from the RCS on two separate occasions; (2) subsequent to the discovery of a leaking valve in the "A-train" RHR heat exchanger, the "B-train" RHR pump was found to be seized.
- The above sequence of events, if not halted in time, could have resulted in loss of the RHR pump (via cavitation). Complications arising from the existence of the gas bubble may have prevented timely restoration of decay heat removal capability.
- The licensee was slow to both effect timely recovery from the above sequence of events and to realize the associated safety implications.
- A number of contributing causes cited by the AIT included: lack of a questioning attitude, poor decision-making, poor equipment condition/lack of adequate instrumentation, poor procedures, inadequate training, and inappropriate planning and scheduling.
- The NRC issued an Information Notice regarding this event, and additional generic action is under consideration.

Mr. P. L'Heureux (Yankee Atomic Electric), who led the licensee's incident response team, made some comments in response to the staff presentation. He noted that while nitrogen use was logged every eight hours, no trending was done. Also, several other activities which made use of nitrogen were underway, thus masking the leakage. The valve that leaked nitrogen had not been used in the history of

the plant (28 years), so its failure to fully close was seen, in retrospect, as quite likely. Containment integrity was intact during the event. Finally, the operators had noticed an upward trend in the pressurizer water level and were investigating this anomaly when the gas leakage into the RCS was discovered.

Committee Discussion

Dr. Kress asked if this event has been factored into the Accident Sequence Precursor program. NRR said it will be so evaluated.

Mr. Barton noted, in summary, that this event contained no surprises; rather, it points to the need for licensees to exercise increased awareness of plant conditions during shutdown operations. This will become more of a challenge, as most licensees plan to extend fuel cycles out to 24 months in response to utility deregulation.

Dr. Powers indicated that the issue of shutdown operation events cries out for use of PRA, which he said is not being applied here. He also stated that he intends to ask the Planning and Procedures Subcommittee to investigate this matter. Dr. Miller indicated that the unambiguous measurement of the reactor vessel water level should be considered a critical parameter to be continuously monitored at all times, similar to the monitoring activities associated with the core neutron flux.

VII. EXECUTIVE SESSION (Open)

[Note: Dr. John T. Larkins was the Designated Federal Official for this portion of the meeting.]

A. Reports, Letters, and Memoranda

Plant-Specific Application of Safety Goals (Report to Shirley Ann Jackson, Chairman, NRC, from T. S. Kress, Chairman, ACRS, dated November 18, 1996)

Position on Direction Setting Issue 22 - Future Role of NRC Research (Report to Shirley Ann Jackson, Chairman, NRC, from T. S. Kress, Chairman, ACRS, dated November 19, 1996)

Proposed Rule on Steam Generator Integrity (Letter to James M. Taylor, Executive Director for Operations, NRC, from T.S. Kress, Chairman, ACRS, dated November 20, 1996)

NRC Programs for Risk-Based Analysis of Reactor Operating Experience (Letter to James M. Taylor, Executive

Director for Operations, NRC, from T. S. Kress, Chairman, ACRS, dated November 22, 1996)

Proposed Final Regulatory Guide Pertaining to the Preparation of Petitions for Rulemaking under 10 CFR 2.802 (Memorandum to James M. Taylor, Executive Director for Operations, NRC, from John T. Larkins, Executive Director, ACRS, dated November 18, 1996) - Consistent with the Committee's decision, Dr. Larkins informed Mr. Taylor that the Committee decided not to review the subject regulatory guide.

Draft Reports Related to the Keowee Hydro Station Emergency Electrical System Supply to the Oconee Nuclear Station (Memorandum to James M. Taylor, Executive Director for Operations, NRC, from John T. Larkins, Executive Director, ACRS, dated November 18, 1996) - Consistent with the Committee's decision, Dr. Larkins informed Mr. Taylor that the Committee decided not to review the subject reports at this time. The Committee, however, may hear a briefing after the licensee has completed the proposed modifications identified in the subject reports.

B. Reconciliation of ACRS Comments and Recommendations

[Note: Mr. Sam Duraiswamy was the Designated Federal Official for this portion of the meeting.]

The Committee discussed the response from the NRC Executive Director for Operations to ACRS comments and recommendations included in a recent ACRS report:

EDO letter dated November 1, 1996, responding to the ACRS letter dated October 23, 1996, concerning the Draft Update of Standard Review Plan, Chapter 7, "Instrumentation and Controls."

The Committee decided that it was satisfied with the EDO response.

C. Report on the Meeting of the Planning and Procedures Subcommittee Held on November 5, 1996 (Open/Closed)

The Committee heard a report from Dr. Kress on the Planning and Procedures Subcommittee meeting held on November 5, 1996. The following items were discussed:

1. ELECTION OF ACRS OFFICERS (Open)

According to the ACRS Bylaws, Members not wishing to hold office for CY 1997 should notify the ACRS Executive Director in writing at least two weeks prior to the election. Since the election is expected to be held on December 6, 1996, the Members should notify the Executive Director by November 22, 1996.

RECOMMENDATION

The Subcommittee recommended that Members notify the Executive Director in writing if they do not wish to hold office for CY 1997.

2. COMMENTS ON DIRECTION-SETTING ISSUES (DSIs) BY ACRS MEMBERS (Open)

ACRS Members who are planning to provide comments on the Commission's Direction Setting Issues (DSI) should provide copies to Dr. El-Zeftawy by Friday, November 8, 1996, so other Members can decide if they want to endorse any of these comments. Comments provided by certain Members were distributed to the Members on Thursday, November 7, 1996.

RECOMMENDATION

The Subcommittee recommended that any Member who had comments on a DSI give them to Dr. El-Zeftawy immediately.

3. FRN AND PRESS RELEASE TO SOLICIT NEW ACRS MEMBERS (Open)

A Press Release and a Federal Register Notice (pp. 1-4) were issued to solicit candidates for membership on the ACRS, with a background in plant operations/risk and thermal-hydraulics/computational fluid dynamics. The closing date is December 31, 1996.

4. FOLLOW-UP FROM ACRS RETREAT (Open)

R. Savio has drafted a list of conclusions and action items from the ACRS Retreat in Cambridge, MA, during October 17-19, 1996. R. Seale will summarize these at this meeting.

RECOMMENDATION

The Subcommittee recommended that a more detailed report on the Retreat be prepared for the December ACRS meeting.

5. INTERNATIONAL ACTIVITIES (Open)

The Quadripartite Meeting is tentatively scheduled for October 20-24, 1997, in the vicinity of Tokyo. A brief report will be given by R. Summers on her meeting with Dr. Sumita in Japan.

RECOMMENDATION

The Subcommittee recommended that no action be taken on the Quadripartite Meeting until a written proposal was received from Dr. Sumita.

6. MEMBERS' ISSUES (Open)

Dr. George Apostolakis requested approval for travel to Kobe, Japan, on March 3-6, 1997.

RECOMMENDATION

The Subcommittee recommended that the trip be approved.

7. STATUS OF ACRS APPOINTMENTS (Open/Closed)

The status of appointments of potential ACRS Members was discussed.

D. Future Meeting Agenda

Appendix IV summarizes the proposed items endorsed by the Committee for the 437th ACRS Meeting, December 5-7, 1996.

The 436th ACRS meeting was adjourned at 2:00 p.m. on Saturday, November 9, 1996.

summarized in NASA's request for
 EOE of Management and Budget
 (OMB) approval.

FOR FURTHER INFORMATION CONTACT:
 Beesie B. Berry, NASA Reports Officer,
 (202) 358-1368.

Reports

Title: NASA Safety Reporting System
 (NSRS).

OMB Number: 2700-0063.

Type of Review: Extension.

Need and Uses: Forms will be used by
 NASA employees and NASA contractor
 employees to voluntarily and
 confidentially report to an independent
 agent any safety concerns or hazards
 pertaining to any NASA program or
 project.

Affected Public: Individuals or
 households, Business or other for-profit,
 Federal Government.

Estimated Number of Respondents:
 75.

Responses Per Respondent: 1.

Estimated Annual Responses: 10.

Estimated Hours Per Request: .25.

Estimated Annual Burden Hours: 19.

Frequency of Report: As required.

Dated: October 18, 1996.

Russell S. Rice,

Director, IRM Division.

9 Doc. 96-27369 Filed 10-24-96; 8:45 a.m.]

LEGAL CODE 7930-01-01

NUCLEAR REGULATORY COMMISSION

Nuclear Safety Research Review Committee

AGENCY: Nuclear Regulatory
 Commission.

ACTION: Notice of meeting.

The Nuclear Safety Research Review
 Committee (NSRRC) will hold its next
 meeting on November 14-15, 1996. The
 location of the meeting will be in Room
 T-10A1, Two White Flint North
 (TWFN) Building, 11545 Rockville Pike,
 Rockville, MD and will be held from
 1:00 p.m. to 5:00 p.m. on the 14th and
 from 9:30 a.m. to 5:30 p.m. on the 15th.

The meeting will be held in
 accordance with the requirements of the
 Federal Advisory Committee Act
 (FACA) and will be open to public
 attendance. The NSRRC provides advice
 to the Director of the Office of Nuclear
 Regulatory Research (RES) on matters of
 overall management importance in the
 direction of the NRC's program of
 nuclear safety research. The main
 purpose of this meeting will be: (1) to
 evaluate the value and contributions of
 the NSRRC in carrying out the NRC's

mission and to develop a set of criteria
 under which the performance of the
 NSRRC could be evaluated in the future;
 (2) to discuss the roles of the NSRRC
 and the Advisory Committee for Reactor
 Safeguards (ACRS) to determine the
 areas of common interest of the two
 Committees; and (3) to discuss potential
 overlap of on-going activities of the
 ACRS and NSRRC Committee and
 coordinate these activities to ensure that
 areas of joint interest are supportive and
 complimentary and not duplicative. As
 time permits, a discussion will be
 initiated on the core technical
 competence to be maintained by the
 NRC's Office of Research staff.

Participants in parts of the discussion
 will include senior NRC staff and other
 RES technical staff as necessary.

Members of the public may file
 written statements regarding any matter
 to be discussed at the meeting. Members
 of the public may also make requests to
 speak at the meeting, but permission to
 speak will be determined by the
 Committee chairperson in accordance
 with procedures established by the
 Committee. A verbatim transcription
 will be made of the NSRRC meeting and
 a copy of the transcript will be placed
 in the NRC's Public Document Room in
 Washington, DC.

Any inquiries regarding this notice or
 any subsequent changes in the status
 and schedule of the meeting, may be
 made to the Designated Federal Officer,
 Dr. Jose Luis M. Cortez (telephone: 301-
 415-6596), between 8:15 a.m. and 5:00
 p.m.

Dated at Rockville, Maryland this 21st day
 of October, 1996.

For the Nuclear Regulatory Commission.

Andrew L. Bates,

Federal Advisory Committee Management
 Officer.

[FR Doc. 96-27412 Filed 10-24-96; 8:45 a.m.]

LEGAL CODE 7930-01-P

* Advisory Committee on Reactor Safeguards; Meeting Notice

In accordance with the purposes of
 Sections 29 and 182b. of the Atomic
 Energy Act (42 U.S.C. 2039, 2232b), the
 Advisory Committee on Reactor
 Safeguards will hold a meeting on
 November 7-9, 1996, in Conference
 Room T-2B3, 11545 Rockville Pike,
 Rockville, Maryland. The date of this
 meeting was previously published in
 the Federal Register on Monday,
 November 27, 1995 (60 FR 58393).

Thursday, November 7, 1996

8:30 a.m.-8:45 a.m.: Opening Remarks
 by the ACRS Chairman

(Open)—The ACRS Chairman will
 make opening remarks regarding
 conduct of the meeting and comment
 briefly regarding items of current
 interest. During this session, the
 Committee will discuss priorities for
 preparation of ACRS reports.

8:45 a.m.-10:45 a.m.: Proposed Rule on
 Steam Generator Integrity

(Open)—The Committee will hear
 presentations by and hold discussions
 with representatives of the NRC staff,
 Nuclear Energy Institute (NEI), and
 Electric Power Research Institute (EPRI)
 regarding the proposed rule on steam
 generator integrity and an associated
 regulatory guide.

Other interested parties will
 participate, as appropriate.

11:00 a.m.-12:15 p.m.: Risk-Based
 Analysis of Reactor Operating
 Experience

(Open)—The Committee will hear
 presentations by and hold discussions
 with representatives of the NRC staff
 regarding the staff activities associated
 with risk-based analysis of reactor
 operating experience, accident sequence
 precursor program, development of risk-
 based performance indicators, and
 related matters.

Representatives of the nuclear
 industry will participate, as appropriate.

1:15 p.m.-3:15 p.m.: Revised Source
 Term for Operating Reactors

(Open)—The Committee will hear
 presentations by and hold discussions
 with representatives of the NRC staff,
 NEI, and Entergy Operations, Inc.,
 regarding the use of revised source term
 for operating plants and the NRC staff's
 proposed approach for reviewing
 applications for license amendments.

Other interested parties will
 participate, as appropriate.

3:30 p.m.-4:30 p.m.: Emergency
 Planning for Advanced Reactors

(Open)—The Committee will hear
 presentations by and hold discussions
 with representatives of the NRC staff
 regarding a simplified approach to
 emergency planning for advanced
 reactors.

Representatives of the nuclear
 industry will participate, as appropriate.

4:30 p.m.-7:00 p.m.: Preparation of
 ACRS Reports

(Open)—The Committee will discuss
 proposed ACRS reports on matters
 considered during this meeting as well

as a proposed report on plant-specific application of Safety Goals.

Friday, November 8, 1996

8:30 a.m.-8:35 a.m.: *Opening Remarks by the ACRS Chairman*

(Open)—The ACRS Chairman will make opening remarks regarding conduct of the meeting.

8:35 a.m.-9:00 a.m.: *Future ACRS Activities*

(Open)—The Committee will discuss recommendations of the Planning and Procedures Subcommittee regarding items proposed for consideration by the full Committee during future meetings.

9:00 a.m.-9:15 a.m.: *Reconciliation of ACRS Comments and Recommendations*

(Open)—The Committee will discuss responses from the NRC Executive Director for Operations (EDO) to comments and recommendations included in recent ACRS reports. The EDO responses are expected to be provided in writing to the ACRS prior to the meeting.

9:15 a.m.-9:45 a.m.: *Report of the Planning and Procedures Subcommittee*

(Open/Closed)—The Committee will hear a report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business, and organizational and personnel matters relating to ACRS.

A portion of this session may be closed to discuss organizational and personnel matters that relate solely to the internal personnel rules and practices of this Advisory Committee, and matters the release of which would constitute a clearly unwarranted invasion of personal privacy.

10:00 a.m.-11:00 a.m.: *Nitrogen Bubble in the Reactor Coolant System at the Haddam Neck Nuclear Power Plant*

(Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the findings and recommendations of the Augmented Inspection Team which investigated the August 28, 1996 event at the Haddam Neck Nuclear Power Plant that involved creation of a nitrogen bubble in the reactor coolant system.

Representatives of the licensee will participate, as appropriate.

11:00 a.m.-12:30 p.m.: *Annual ACRS Report to Congress*

(Open)—The Committee will discuss the format and content of the annual ACRS report to Congress on the NRC Safety Research Program.

1:30 p.m.-7:00 p.m.: *Preparation of ACRS Reports*

(Open)—The Committee will continue its discussion of the proposed ACRS reports on matters considered during this meeting as well as a proposed report on plant-specific application of Safety Goals.

Saturday, November 9, 1996

8:30 a.m.-12:30 p.m.: *Preparation of ACRS Reports*

(Open)—The Committee will continue discussion of proposed ACRS reports on matters considered during this meeting as well as a proposed report on plant-specific application of Safety Goals.

12:30 p.m.-1:30 p.m.: *Strategic Planning*

(Open)—The Committee will continue its discussion of items of significant importance to NRC, including reestablishing of the Committee activities for FY 97.

Procedures for the conduct of and participation in ACRS meetings were published in the Federal Register on October 1, 1996 (61 FR 51310). In accordance with these procedures, oral or written statements may be presented by members of the public, electronic recordings will be permitted only during the open portions of the meeting, and questions may be asked only by members of the Committee, its consultants, and staff. Persons desiring to make oral statements should notify Mr. Sam Duraiswamy, Chief, Nuclear Reactors Branch, at least five days before the meeting, if possible, so that appropriate arrangements can be made to allow the necessary time during the meeting for such statements. Use of still, motion picture, and television cameras during this meeting may be limited to selected portions of the meeting as determined by the Chairman. Information regarding the time to be set aside for this purpose may be obtained by contacting the Chief of the Nuclear Reactors Branch prior to the meeting. In view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with the Chief of the Nuclear Reactors Branch if such rescheduling would result in major inconvenience.

In accordance with Subsection 10(d) P.L. 92-463, I have determined that it is necessary to close portions of this meeting noted above to discuss matters that relate solely to the internal personnel rules and practices of this Advisory Committee per 5 U.S.C. 552b(c)(2), and to discuss matters the release of which would constitute a

clearly unwarranted invasion of personal privacy per 5 U.S.C. 552b(c)(6).

Further information regarding topics to be discussed, whether the meeting has been cancelled or rescheduled, the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor can be obtained by contacting Mr. Sam Duraiswamy, Chief, Nuclear Reactors Branch (telephone 301/415-7364), between 7:30 a.m. and 4:15 p.m. EDT.

ACRS meeting notices, meeting transcripts, and letter reports are now available on FedWorld from the "NRC MAIN MENU." Direct Dial Access number to FedWorld is (800) 303-9672 or ftp.fedworld. These documents and the meeting agenda are also available for downloading or reviewing on the internet at <http://www.nrc.gov/ACRSACNW>.

Dated: October 22, 1996.

Andrew L. Bates,
Advisory Committee Management Officer.
[FR Doc. 96-27413 Filed 10-24-96; 8:45 am]
BILLING CODE 7005-01-P

SECURITIES AND EXCHANGE COMMISSION

[Rel. No. IC-22291; 812-10216]

First Trust Special Situations Trust and Nike Securities L.P.; Notice of Application

October 21, 1996.

AGENCY: Securities and Exchange Commission ("SEC").

ACTION: Notice of application for exemption under the Investment Company Act of 1940 (the "Act").

APPLICANTS: First Trust Special Situations Trust and Nike Securities L.P.

RELEVANT ACT SECTIONS: Order requested under section 6(c) of the Act for an exemption from section 12(d)(1)(F)(ii) of the Act.

SUMMARY OF APPLICATION: Applicants request an order that would permit series of the Trust (each a "Series" or "Trust Series"), to offer units to the public with a sales load that exceeds the 1.5% sales load limitation of section 12(d)(1)(F)(ii) of the Act.

FILING DATE: The application was filed on June 24, 1996 and amended on September 5, 1996.

HEARING OR NOTIFICATION OF HEARING: An order granting the application will be issued unless the SEC orders a hearing. Interested persons may request a hearing by writing to the SEC's Secretary and serving applicants with a

APPENDIX II



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

October 21, 1996

SCHEDULE AND OUTLINE FOR DISCUSSION
436th ACRS MEETING
NOVEMBER 7-9, 1996

THURSDAY, NOVEMBER 7, 1996, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH,
ROCKVILLE, MARYLAND

1) 8:30 - 8:45 A.M.

Opening Remarks by the ACRS Chairman (Open)

- 1.1) Opening Statement (TSK/SD)
- 1.2) Items of current interest
(TSK/JTL/SD)
- 1.3) Priorities for preparation of ACRS
reports (TSK/SD)

2) 8:45 - 10:⁵~~45~~ A.M.

Proposed Rule on Steam Generator Integrity
(Open) (RLS/NFD)

- 2.1) Remarks by the Acting Subcommittee
Chairman
- 2.2) Briefing by and discussions with
representatives of the NRC staff,
Nuclear Energy Institute (NEI),
and Electric Power Research Institute
(EPRI) regarding the proposed rule
on steam generator integrity and an
associated regulatory guide.

Other interested parties will participate,
as appropriate.

10:⁵~~45~~ - 11:¹³~~00~~ A.M.

BREAK

3) 11:¹³~~00~~ - 12:²⁵~~15~~ P.M.

Risk-Based Analysis of Reactor Operating
Experience (Open) (GA/MTM)

- 3.1) Remarks by the Subcommittee Chairman
- 3.2) Briefing by and discussions with
representatives of the NRC staff
regarding the staff activities
associated with risk-based analysis
of reactor operating experience,
accident sequence precursor program,
development of risk-based performance
indicators, and related matters.

Representatives of the nuclear industry will
participate, as appropriate.

{ TRANSCRIBED PORTIONS OF THE MEETING }

12:15²⁵ - 1:15³⁰ P.M.

LUNCH

4) 1:15³⁰ - 3:15^{2:03} P.M.

Revised Source Term for Operating Reactors

(Open) (MHF/AS)

- 4.1) Remarks by the Subcommittee Chairman
- 4.2) Briefing by and discussions with representatives of the NRC staff, NEI, and Entergy Operations Inc., regarding the use of revised source term for operating plants and the NRC staff's proposed approach for reviewing applications for license amendments.

Other interested parties will participate, as appropriate.

ACRS Reports

BREAK

2:03 - 3:00 P.M.

3:15⁰⁰ - 3:30²⁰ P.M.

5) 3:30²⁶ - 4:30 P.M.

Emergency Planning for Advanced Reactors

(Open) (RLS/MME)

- 5.1) Remarks by the Subcommittee Chairman
- 5.2) Briefing by and discussions with representatives of the NRC staff regarding a simplified approach to emergency planning for advanced reactors.

Representatives of the nuclear industry will participate, as appropriate.

BREAK

4:30 - 4:50 P.M.

6) 4:30⁵⁰ - 7:00^{6:40} P.M.

Preparation of ACRS Reports (Open)

Discussion of proposed ACRS reports on:

- 6.1) Proposed Rule on Steam Generator Integrity and an associated Regulatory Guide (RLS/NFD)
- 6.2) Risk-Based Analysis of Reactor Operating Experience (GA/MTM)
- 6.3) Revised Source Term for Operating Reactors (MHF/AS)
- 6.4) Plant-Specific Application of Safety Goals (TSK/NFD)

FRIDAY, NOVEMBER 8, 1996, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH,
ROCKVILLE, MARYLAND

7) 8:30 - 8:35 A.M. Opening Remarks by the ACRS Chairman (Open)
(TSK/SD)

8) 8:35 - 9:00 A.M. Future ACRS Activities (Open) (TSK/SD)
Discussion of the recommendations of the Planning and Procedures Subcommittee regarding items proposed for consideration by the full Committee during future meetings.

9) 9:00 - 9:⁰¹~~15~~ A.M. Reconciliation of ACRS Comments and Recommendations (Open) (TSK, et.al./SD, et.al.)
Discussion of the responses from the NRC Executive Director for Operations to comments and recommendations included in recent ACRS reports.

10) 9:⁰²~~15~~ - 9:⁵⁰~~45~~ A.M. Report of the Planning and Procedures Subcommittee (Open/Closed) (TSK/JTL)
Report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business, and organizational and personnel matters relating to the ACRS.

[Note: A portion of this session may be closed to discuss organizational and personnel matters that relate solely to the internal personnel rules and practices of this Advisory Committee, and matters the release of which would constitute a clearly unwarranted invasion of personal privacy.]

9:⁵⁰~~45~~ - 10:⁰⁵~~00~~ A.M.

BREAK

11) 10:⁰⁵~~00~~ - 11:⁰³~~00~~ A.M.

Nitrogen Bubble in the Reactor Coolant System at the Haddam Neck Nuclear Power Plant (Open) (JJB/PAB)

11.1) Remarks by the Subcommittee Chairman

- 11.2) Briefing by and discussions with representatives of the NRC staff regarding the findings and recommendations of the Augmented Inspection Team which investigated the August 28, 1996 event at the Haddam Neck Nuclear Power Plant that involved creation of a nitrogen bubble in the reactor coolant system.

Representatives of the licensees will participate, as appropriate.

12) 11:00³ - 12:00⁰⁵ P.M.

Annual ACRS Report to Congress (Open)
(RLS/MME)

Discussion of the format and content of the annual ACRS report to Congress on the NRC Safety Research Program.

12:00⁰⁵ - 1:00²² P.M.

LUNCH

13) 1:00²² - 7:00^{6:30} P.M.
(3:30-3:45 BREAK)
10 20

Preparation of ACRS Reports (Open)

Discussion of proposed ACRS reports on:

- 13.1) Proposed Rule on Steam Generator Integrity and an associated Regulatory Guide (RLS/NFD)
- 13.2) Risk-Based Analysis of Reactor Operating Experience (GA/MTM)
- 13.3) Revised Source Term for Operating Reactors (MHF/AS)
- 13.4) Plant-Specific Application of Safety Goals (TSK/NFD)

SATURDAY, NOVEMBER 9, 1996, CONFERENCE ROOM 2B3, TWO WHITE FLINT NORTH, ROCKVILLE, MARYLAND

14) 8:30⁵ - 12:30 P.M.

Preparation of ACRS Reports (Open)

Continue discussion of proposed ACRS reports listed under Item 13.

10:15 - 10:30 P.M.

BREAK

15) 12:30 - 1:00^{2:00} P.M.

Strategic Planning (Open) (TSK/JTL)

Discussion of items of significant importance to NRC, including rebaselining of the Committee activities for FY 97

- NOTE:
- Presentation time should not exceed 50 percent of the total time allocated for a specific item. The remaining 50 percent of the time is reserved for discussion.
 - Number of copies of the presentation materials to be provided to the ACRS - 35.

APPENDIX III: MEETING ATTENDEES

436TH ACRS MEETING
NOVEMBER 7-9, 1996

NRC STAFF

C. Ader	RES
E. Benner	NRR
R. Borchardt	NRR
A. Chaffee	NRR
T. Collins	NRR
S. Dembele	NRR
K. Desai	NRR
J. Donoghue	NRR
R. Emch	NRR
T. Essig	NRR
J. Flack	NRR
E. Goodwin	OECB/NRR
J. Hayes	NRR
M. Hart	NRR
R. Herman	NRR
W. Hodges	RES
J. Hopkins	NRR
A. Huffert	NRR
J. Hyslop	NRR
C. Jackson	NRR
R. Jones	NRR
F. Kantor	NRR
A. Levin	NRR
S. Long	NRR
W. Lyon	NRR
T. Martin	NRR
C. Miller	NRR
J. Mitchell	OEDO
J. Morrison	RES
J. Munroe	NRR
J. Muscara	RES
J. O'Brien	NRR
K. Parczewski	NRR
B. Palla	NRR
T. Quay	NRR
D. Rasmuson	AEOD
T. Reed	NRR
J. Ridgely	RES
C. Rossi	AEOD
J. Schaperow	RES
L. Shao	NRR
J. Wilson	NRR
J. Wilson	NRR

ATTENDEES FROM OTHER AGENCIES AND GENERAL PUBLIC

M. Beaumont	Westinghouse
C. Callaway	NEI
T. Cleary	Connecticut Yankee - Haddam Neck
K. Cozens	NEI
K. Green	Sciencetech
C. Inelty	EPRI
T. Kevortian	FENA
P. L'Heureux	Connecticut Yankee - Haddam Neck
D. Leaver	Polestar
J. Metcalf	Polestar
A. Nelson	NEI
R. Pearson	NSP
F. Quinn	NUS Information Services
R. Simard	NEI
C. Smith	BGE
T. Zana	TEPCO

APPENDIX IV: FUTURE AGENDA

The Committee agreed to consider the following during the 437th ACRS Meeting, December 5-7, 1996:

- 1) 12:00 - 12:05 P.M. ELECTION OF ACRS OFFICERS (Open) (JTL)

A list of members who have withdrawn their names from being considered as candidates for ACRS officers is attached (p. 1). Elections will be held during this meeting.

- 2) 12:05 - 12:10 P.M. Conflict-of-Interest Procedures (Open)
(SD)

Procedures have been drafted to ensure that members have a consistent policy covering conflict-of-interest requirements during meetings (pp. 2-4). These procedures have been sent to OCC for review. Members are invited to comment.

- 3) 12:10 - 12:15 P.M. FEDERAL REGISTER NOTICE AND PRESS RELEASE
TO SOLICIT CANDIDATES FOR
APPOINTMENT TO THE ACRS
(Open) (JTL)

The closing date for applications for appointment to the ACRS has been postponed from December 31, 1996 to January 31, 1997.

- 4) 12:15 - 12:25 P.M. COMMITTEE ROLE IN REVIEWING GENERIC
LETTERS (Open) (SD)

Certain safety-significant generic letters have been approved on an exigent basis without ACRS review (pp. 5-7). The Committee should determine its role in reviewing such letters. The MOU states that any deviation from the MOU should be mutually agreed to by the EDO and the ACRS Executive Director.

- 5) 12:25 - 12:35 P.M. INTERNATIONAL ACTIVITIES (Open) (RS)

An email message has been received from Mr. Yamada, of the Science and Technology Agency of Japan (pp. 8-10). The Committee should review the suggested dates, topics for discussion, and sites for a trip visit and indicate a

preference for transmittal to Mr. Yamada.

6) 12:35 - 12:45 P.M. MEETING WITH CHAIRMAN JACKSON AND
ANNOUNCEMENTS (Open) (JTL)

The Executive Director met with Chairman Jackson alone twice and together with Dr. Apostolakis once since the last ACRS meeting. He will brief the Committee on these discussions as well as on recent announcements by Chairman Jackson regarding the direction of the agency.

7) 12:45 - 12:50 P.M. TIMING OF RESPONSE TO SRM ACTION
ITEMS (Open) (SD)

A review has been made of Staff Requirements Memoranda over the past 36 months and it has been determined that the ACRS was not late in responding to these action items.

8) 12:50 - 12:55 P.M. CRITERIA TO EVALUATE ACRS CANDIDATES
(Open) (JTL)

The ACRS staff has been asked to draft criteria to determine the eligibility of candidates (principally ex-utility officials) for ACRS membership. Mr. Jay Carroll has been asked to assist in this effort, which will also be coordinated with OGC.

9) 12:55 - 1:05 P.M. ASSESSMENT OF TECHNICAL EXPERTISE OF ACRS
MEMBERS (Open) (SD)

The staff has been asked to provide a list of the areas of technical expertise of current ACRS members and an assessment of future needs of the Committee, based on current capabilities, future vacancies, and most importantly, the projected workload and topics for review.

10) 1:05 - 1:15 P.M. NSRRC PARTICIPATION IN ACRS MATTERS

(Open) (RPS)

It is anticipated that the NSRRC and ACRS members will be on occasion attending ACRS or, in the case of the ACRS, NSRRC

meetings. The procedures described in the attachment (pp. 11-27) are proposed and will be sent to Dr. Morrison as a proposal after ACRS comment.

11) 1:15 - 1:25 P.M. MEMBERS ISSUES

Dr. Robert Seale has requested ACRS support for a trip to France to participate in a review of recent Phebus experiments (see p. 28)

11) 1:25 - 1:45 P.M. FUTURE ACTIVITIES (Open) (SD)

[See separate handout]

APPENDIX V
LIST OF DOCUMENTS PROVIDED TO THE COMMITTEE

[Note: Some documents listed below may have been provided or prepared for Committee use only. These documents must be reviewed prior to release to the public.]

MEETING HANDOUTS

AGENDA
ITEM NO.

DOCUMENTS

1 Opening Remarks by the ACRS Chairman

1. Introductory Statement of the ACRS Chairman, dated November 7-9, 1996 [Handout]
2. Items of Interest [Handout]
3. Notice concerning Jocelyn Mitchell (undated) [Handout]
4. Email message from Center Network Control, dated November 5, 1996: Firm Time for RES Earthquake Seminar [Handout]

2 Proposed Rule on Steam Generator Integrity

5. Proposed Steam Generator Integrity Rule: Differing Professional Opinion, dated November 7, 1996, presented by Dr. Joe Hopenfeld, Engineer, RES [Viewgraphs]
6. Proposed Steam Generator Rule Making, dated November 7, 1996, presented by Jack Strosneider and Joe Donoghue, NRR [Viewgraphs]
7. Proposed Steam Generator Rule: Safety Consideration, dated November 7, 1996, presented by Joseph Donoghue, NRR [Viewgraphs]
8. Industry Perspective, dated November 7, 1996, presented by Richard Pearson [Viewgraphs]
9. Proposed Rule on Steam Generator Integrity (related documents), November 7, 1996 [Handout #2.1]
10. Memorandum from J. Hopenfeld, RES, to E. Beckjord, Director, RES, dated September 11, 1992: Addendum to March 27, 1992 Memo Regarding Degraded Steam Generator Tubes [Handout #2.2]

3 Risk-Based Analysis of Reactor Operating Experience

11. Risk-Based Analysis of Reactor Operating Experience,

dated November 7, 1996, presented by Patrick W. Baranowsky, AEOD [Viewgraphs]

5 Emergency Planning for Advanced Reactors

12. ACRS Briefing on NRC's Evaluation of Technical Criteria and Methods Which Could Be Used to Justify Simplification of Emergency Preparedness for Reactors with Greater Safety Margins, presented by James O'Brien, NRR [Viewgraphs]

13. Emergency Response Planning Appropriate to Advanced Reactor Designs, dated November 7, 1996, presented by Ron Simard, Nuclear Energy Institute [Viewgraphs]

6 Preparation of ACRS Reports

14. Direction-Setting Issues (DSIs): ACRS Members' Comments, authors: Dr. Dana Powers, Dr. Mario Fontana, and Mr. John Barton [Handout # 6.1]

8 Future ACRS Activities

15. Future ACRS Activities - 437th ACRS Meeting, December 5-7, 1996 [Handout #8-1]

9 Reconciliation of ACRS Comments and Recommendations

16. Reconciliation of ACRS Comments and Recommendations [Handout #9.1]

10 Report of the Planning and Procedures Subcommittee

17. Final Draft Minutes of Planning and Procedures Subcommittee Meeting - November 5, 1996 [Handout #10.1]

11 Nitrogen Bubble in the Reactor Coolant System at the Haddam Neck Nuclear Power Plant

18. Haddam Neck: Decay Heat Removal System Challenges, dated August 28, 1996, presented by James Trapp, Region I [Viewgraphs]

MEETING NOTEBOOK CONTENTS

TAB

DOCUMENTS

- 2 Proposed Rule on Steam Generator Integrity
 1. Table of Contents
 2. Proposed Agenda
 3. Status Report
 4. Memorandum from N. Dudley, ACRS Staff Engineer, to ACRS Members: Certification of the Minutes of the ACRS Joint Subcommittee Meeting on Materials & Metallurgy and Severe Accidents, dated July 22, 1996.
 5. Memorandum from B. Sheron, NRR, to J. Larkins, ACRS Executive Director: ACRS Review of the Proposed Steam Generator Rule, dated October 25, 1996.

- 3 Risk-Based Analysis of Reactor Operating Experience
 6. Table of Contents
 7. Proposed Schedule
 8. Project Status Report
 9. Memorandum from M. Markley, ACRS Staff Engineer, to P. Baranowsky, AEOD: Comments and Questions by Individual ACRS Members for Use by AEOD in Preparing for the Next PRA Subcommittee Meeting, dated October 30, 1996.
 10. Program Guide from S. Mays, RRAB, to C. Rossi, SPD: Risk-Based Analysis of Reactor Operating Experience, dated December 15, 1995.

- 4 Revised Source Term for Operating Reactors
 11. Table of Contents
 12. Proposed Schedule
 13. Status Report
 14. Report from W. Lindblad, ACRS Member, to I. Selin, NRC Chairman: Proposed Final Version of NUREG-1465 "Accident Source Terms for Lightwater Nuclear Power Plants", dated September 20, 1994.
 15. Memorandum from J. Taylor, Executive Director for Operations, to S.A. Jackson, NRC Chairman: Use of NUREG-1465 Source Term at Operating Reactors, dated August 9, 1996.
 16. Memorandum from J. Hoyle, NRC Secretary, to J. Taylor, Executive Director for Operations: Briefing on Part 100 Final Rule on Reactor Site Criteria, dated July 2, 1996.
 17. Draft SECY from J. Taylor, Executive Director for Operations, to S. A. Jackson, NRC Chairman: Use of NUREG-1465 Source Term at Operating Reactor.
 18. Note from J. Williams, Project Manager, to F. Hebdon, Director, Director, Project Directorate II-3, NRR:

Application of New source Term Methodology to Support Increased Main Steam Isolation Valve Leakage at Browns Ferry, dated December 29, 1995

19. Report: Revised Radiological Consequences of Accidents for Indian Point Unit 2 Taking into Account Source Term Methodology from NUREG-1465, dated July 1996.

5 Emergency Planning for Advanced Reactors

20. Table of Contents
21. Proposed Sechedule
22. Status Report
23. Memorandum from J. Taylor, Executive Director for Operations, to S. Chilk, Secretary: SECY 93-092 - Issues Pertaining to the Advanced Reactor (Prism MHTGR, and Pius) and Candu 3 Designs and Their Relationship to Current Regulatory Requirements, dated July 30, 1993.
24. Letter from T.S. Kress, ACRS Chairman, to I. Selin, NRC Chairman: Emergency Planning Zones, Protective Action Guidelines, and the New Source Terms, dated July 13, 1994.
25. Working Paper: Summary of Staff's Evaluation of Technical Criteria and Methods Which Could Be Used to Justify Simplification of Emergency Preparedness for Reactor with Greater Safety Margins, dated October 7, 1996.

11 Nitrogen Bubble in the Reactor Coolant System at the Haddam Neck Nuclear Power Plant

26. Table of Contents
27. Presentation Schedule
28. Project Status Report
29. NRC Information Notice 94-36, Supplement 1: Undetected Accumulation of Gas in Reactor Coolant System, dated November 1996.
30. Letter from T. Feigenbaum, Connecticut Yankee Atomic Power Company, from H. Miller, Region I Office: Augmented Inspection Team Report Number 50-213/96-80, dated October 30, 1996.