NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Brunswick Steam Electric Plant Unit 1	0 5 0 0 0 3 2 5	1 OF 0 2

TITLE (4) Inadequate Response Time Testing of Main Steam Line Tunnel Temperature Containment Isolation Instrumentation Logic Circuitry

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 9	0 5	8 5	8 5	0 4 9	0 0	1 0	0 3	8 5	Brunswick Unit 2	0 5 0 0 0 3 2 4
										0 5 0 0 0

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
5	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0 0 0	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
M. J. Pastva, Jr., Regulatory Technician	AREA CODE 9 1 9 4 5 7 - 2 3 1 5

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 9-5-85, it was identified plant procedures did not adequately response time test certain relays of the primary containment isolation instrumentation logic. The testing requirement is reflected by Technical Specifications Table 3.3.2-3, Item 1.D. The subject response time tests are for the main steam line tunnel temperature switches (Periodic Test PT-45.2.1) and primary containment isolation (PCI) valves circuit (PT-45.2.6). The untimed logic supports the PCI System exclusively.

The procedural problems apply to Units 1 and 2 and were discovered during a review and rewriting of plant maintenance surveillance procedures. Unit 1 was in a refuel maintenance outage and preparations were underway for startup of Unit 2. The procedural problems are attributed to insufficient technical review during original procedural development.

On 9-7-85, the subject logic circuitry was satisfactorily tested using a plant special procedure. As previously reported in LERs 1-85-020 and 1-85-046, system test descriptions will be developed and implemented by 12-31-85 for response time tests to provide enhanced definition and control of procedural overlap points.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Brunswick Steam Electric Plant Unit 1	0 5 0 0 0 3 2 5	8 5	— 0 4 9	— 0 0	0 2 OF 0 2

TEXT *(If more space is required, use additional NRC Form 366A's)* (17)

On September 5, 1985, it was identified that plant procedures, to response time test primary containment isolation instrumentation, did not adequately test certain relays in the logic. The main steam line tunnel temperature switch response time test (Periodic Test PT-45.2.1) and the primary containment isolation valves circuitry response time test, PT-45.2.6, did not test steam leak detection relays TR/3A-3D. This testing requirement is reflected by Technical Specification Table 3.3.2-3, Item 1.D. The procedural problems apply to Units 1 and 2 and were discovered during review and rewriting of plant maintenance surveillance procedures. Unit 1 was in a refueling/maintenance outage and preparations were underway for startup of Unit 2.

The subject procedural problems result from insufficient technical review during the original procedure development. On September 7, 1985, the subject logic circuitry was tested via plant Special Procedure SP-85-088 and found to be functioning within technical specification response time requirements on both units. Surveillance tests will be revised and/or generated by October 31, 1985, to require periodic response time testing of the primary containment isolation associated with the main steam line tunnel temperature high trip isolation function on a periodic basis.

The ongoing review and rewriting of plant maintenance surveillance procedures also identified prior similar procedural problems which were reported in LERs 1-85-003, 1-85-020, and 1-85-046. The corrective action to these events is embodied within the Maintenance Surveillance Test (MST) procedure rewrite program, scheduled for completion in 1985. By December 31,1985, system test descriptions relative to the MST program will also be developed and implemented for plant surveillance test procedures to provide enhanced definition and control of procedural overlap points.

CP&L
Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429
October 3, 1985

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SERIAL: BSEP/85-1746

NRC Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
LICENSEE EVENT REPORT 1-85-049

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

MJP/mbh

Enclosure

cc: Dr. J. N. Grace

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