



Point Beach Nuclear Plant  
6610 Nuclear Rd., Two Rivers, WI 54241

(414) 755-2321

NPL 97-228

May 5, 1997

Document Control Desk  
US NUCLEAR REGULATORY COMMISSION  
Mail Station P1-137  
Washington, DC 20555

Gentlemen:

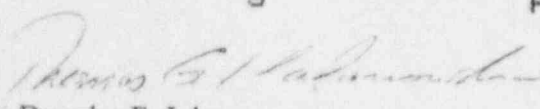
DOCKET 50-266 AND 50-301  
LICENSEE EVENT REPORT 97-018-00  
POTENTIAL RESIDUAL HEAT REMOVAL SYSTEM  
OVERPRESSURE DURING ACCIDENT CONDITIONS  
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

Enclosed is Licensee Event Report 97-018-00 for Point Beach Nuclear Plant, Units 1 and 2. This report is provided in accordance with 10 CFR 50.73(a)(2)(i), "Any event or condition that alone could have prevented the fulfillment of the safety function of structures or systems that are needed to mitigate the consequences of an accident." This report describes a potential to overpressurize the isolated section of pipe between two residual heat removal (RHR) valves inside each containment, during a design basis accident. Any significant damage to the affected pipe or valves could have inhibited the RHR system's capability to achieve and maintain a cold shutdown condition, which is required for certain accidents. This condition is a latent characteristic of the original plant design.

If you require additional information, please contact us.

Sincerely,

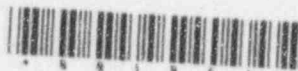
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Douglas F. Johnson  
Manager  
Regulatory Services and Licensing

Enclosure

cc: NRC Resident Inspector  
NRC Regional Administrator

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