



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

NOV 8 1977

11/9 R. Tedesco

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I would like to review a suggestion for creation of a new category of information by Nov. 23.

MEMORANDUM FOR: R. L. Tedesco, Assistant Director for Plant Systems
Division of Systems Safety

FROM: D. G. Eisenhut, Assistant Director for Operational
Technology, Division of Operating Reactors

SUBJECT: 10 CFR PART 21 NOTIFICATION BY GENERAL ELECTRIC COMPANY
REGARDING SAFETY-RELIEF VALVE DISCHARGE LOADS

R-71

your
attach
draft
memo
to
NRR
for
info

On October 18, 1977 OI&E formally transferred the lead responsibility for followup actions associated with the subject notification to NRR. A copy of the "Transfer of Lead Responsibility" agreement is enclosed for your information.

As we have discussed, we believe that the responsibility for the long-term generic review of safety-relief valve (SRV) loads for application to the Mark I, Mark II, and Mark III BWR containment system designs appropriately belongs to DSS. Accordingly, we believe that the lead responsibility for the long-term resolution of this particular concern, with respect to load definition, resides within DSS and that it should be addressed within the scope of the Task Action Plan for NRR Category A Technical Activity No. A-39. Similarly, we believe that the responsibility for the structural evaluations associated with this long-term review resides within DOR for affected operating facilities and within DSS for affected facilities which are not yet licensed for operation.

On the other hand, DOR will continue to perform a short-term evaluation of the new information as it relates to continued safe operation of operating BWRs with the Mark I containment systems to ensure continuing safety while the long-term review is being conducted.

Since this action item will be tracked and reported as a "Transfer of Lead Responsibility from IE to NRR" and as a Part 21 review, it will from time to time be necessary to establish scheduler information for this review. We will be in contact with you as the need for such information arises.

D. G. Eisenhut
D. G. Eisenhut, Assistant Director
for Operational Technology
Division of Operating Reactors

Enclosure:
As stated

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PDR FOIA
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Date: OCT 19 1977

Serial No.: IE-RCI-77-01

TRANSFER OF LEAD RESPONSIBILITY

To: E. G. Case, Acting Director, NRR

Subject: GE RELIEF VALVE CONTROL SYSTEM ANALYSIS

Responsible IE Individual: N. C. Moseley

Description of Item Requiring Resolution:

By letter dated October 11, 1977, General Electric Company (GE), reported to IE, pursuant to 10 CFR 21, the identification of a design condition less conservative than originally predicted. The condition is described in the above letter and its enclosure (copy attached). The condition relates to control of reactor coolant system pressure transients resulting from at-power closure of BWR mainsteam isolation valves. The condition is reported to be generic to BWRs both in operation and under construction.

Recommendations and Proposed Course of Action:

1. The Office of Nuclear Reactor Regulation (NRR) will determine, on a generic basis, the significance of the reported situation, and the adequacy of the proposed corrective action.
2. NRR will modify individual licenses as appropriate.
3. The Office of Inspection and Enforcement (IE) will inspect licensees for compliance to any requirements that may be imposed by NRR.

Concurrence?

Norman C. Moseley
Norman C. Moseley, Director, Division of Reactor Construction Inspection, IE 10/11/77
Date

Edson G. Case
Edson G. Case, Acting Director, Office of Nuclear Reactor Regulation 10/11/77
Date

Enclosure:

G.E. ltr dated Oct. 11, 1977 w/encl.

cc: R. S. Boyd, NRR
V. Stello, NRR
K. R. Goller, NRR
D. G. Eisenhut, NRR
S. A. Varga, NRR
W. F. Kane, NRR
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