



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DEC 13 1977

MEMORANDUM FOR: T. M. Su, Task Manager (A-39), Containment
Systems Branch, DSS

FROM: W. R. Butler, Chief, Plant Systems Branch,
DOR

SUBJECT: COMMENTS ON REVISION 1 TO TASK A-39

We have reviewed your proposed Revision 1 to Category A Task A-39, "Determination of Safety/Relief Valve (SRV) Pool Dynamic Loads and Temperature Limits for BWR Containments". As a result of our review, we recommend the following changes to the proposed revision:

- (1) The discussion of the application of quencher devices to the Mark II plants on page 4a needs clarification. The Mark II owners' alternative approaches for quencher application should be more clearly described.
- (2) The fourth sentence on page 4a is confusing and should be deleted.
- (3) Task C.1.c.(3) on page 7 should be clarified as "SRV design load cases".
- (4) Task C.2.c on page 9 should be modified to reflect the 1/4 scale "T" quencher tests in the Mark I LTP.
- (5) Task 2c on page 15 should be modified to include the 1/4 scale "T" quencher tests.
- (6) The SRV program schedule should be modified to reflect the following Mark I milestones, as documented in revision 2 to the Program Action Plan:

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Task 2.c - Mark I quencher evaluation

a. Small Scale Test Report	June 1978
b. Monticello Quencher Test Report	July 1978
c. SRV Discharge Steam Mixing Model	Jan. 1978
d. SRV Torus Load Definition	Sept. 1978
e. SRV Pipe Load Definition	Oct. 1978
f. FSI Test Report	Aug. 1978
g. FSI Analytical Model	Sep. 1978

Task 3.0 - Submerged Structures

a. LOCA Analytical Model	Jan. 1978
b. Simple Geometry Test Report	May 1978
c. Quencher Analytical Model	Aug. 1978
d. Data Evaluation Report	Dec. 1978



W. R. Butler, Chief
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