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Director
Office of Nuclear Reactor Regulation
US Nuclear Regulatory Commission
Washington, DC 20555

Prairie Island Nuclear Generating Plant
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

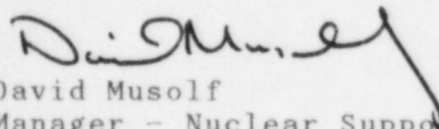
Supplemental Information Related to Response to
Generic Letter 83-28, Salem ATWS Event Generic Items

The purpose of this letter is to supplement the information provided in item 1.1 of our letter dated November 4, 1983. This information was provided earlier to the Prairie Island Project Manager in the Division of Licensing in a conference telephone call with the NRR reviewer for this matter.

Prairie Island evaluates the plant response to a reactor trip to determine if the response was within acceptable limits. Plant administrative work instruction (AWI) 5AWI 3.1.1, "Return to Power After Reactor Trip," requires that this evaluation be done prior to restart. The Sequence of Events Log and the Post Trip Review Listing are used in making this determination. These two records are described in section 1.2 of our November 4, 1983 letter. In addition, 5AWI 3.6.1, "Investigative Reports for Reportable Events," requires the event investigator to describe, analyze, and evaluate the safety implications of the event. This AWI addresses a longer term, in-depth, analysis which is typically completed within 30 days of the event.

We have in place a systematic safety assessment program. This program is specified in 5AWI 3.6.1. This AWI also requires the implementation of 5AWI 3.6.1.

Please contact us if you have any questions related to the information we have provided.


David Musolf
Manager - Nuclear Support Services

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