

April 28, 1997

Southern Nuclear Operating Company, Inc.
ATTN: Mr. C. K. McCoy
Vice President
Vogtle Electric Generating Plant
P. O. Box 1295
Birmingham, AL 35201-1295

SUBJECT: MEETING SUMMARY - VOGTLE ELECTRIC GENERATING PLANT
SELF-ASSESSMENT - DOCKET NOS. 50-424 AND 50-425

Gentlemen:

This refers to the meeting conducted at your request at the NRC Region II Office on April 21, 1997 at 10:00 am. The meeting's purpose was to discuss plant performance during the past SALP cycle for your Vogtle facility. Enclosed are a list of attendees and the presentation handouts.

In accordance with Section 2.790 of the NRC's "Rules of Practice," Part 2, Title 10 Code of Federal Regulations, a copy of this letter and its enclosures will be placed in the NRC Public Document Room.

Should you have any questions concerning this meeting, please contact me at (404) 562-4520.

Sincerely,

(Original signed by Pierce H. Skinner)

Pierce Skinner, Chief
Reactor Projects Branch 2
Division of Reactor Projects

Docket Nos.: 50-424, 50-425
License Nos.: NPF-68, NPF-81

Enclosures:

1. List of Attendees
2. Vogtle Performance Graphs
3. Vogtle Performance Update

cc w/encls: (See Page 2)

9705060210 970428
PDR ADOCK 05000424
G PDR



JE45

cc w/encls:

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Executive Vice President
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J. B. Beasley
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Burke County Commission
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cc w/encls cont'd: (See Page 3)

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Distribution w/encls:

P. Skinner, RII
R. Carrion, RII
W. P. Kleinsorge, RII
M. E. Ernstes, RII
L. Wheeler, NRR
PUBLIC

Distribution w/encls cont'd: (See Page 4)

SNC

4

Distribution w/encls: Continued
 NRC Senior Resident Inspector
 U.S. Nuclear Regulatory Commission
 8805 River Road
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OFFICE	RII:DRP	RII:DRP				
SIGNATURE	<i>ME</i>	<i>PH</i>				
NAME	MEErnstes: dka	PHSkinner				
DATE	04 / 20 / 97	04 / 20 / 97	04 / / 97	04 / / 97	04 / / 97	04 / / 97
COPY?	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO

OFFICIAL RECORD COPY

DOCUMENT NAME: A MEETING.VOG

LIST OF ATTENDEES

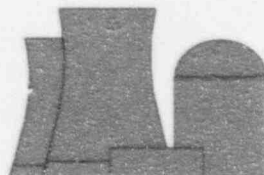
NRC

H. Berkow, NRR
C. Casto, DRS
M. Ernstes, DRP
J. Jaudon, Director DRS
C. Ogle, VEGP Senior Resident Inspector
L. Reyes, Regional Administrator
W. Rogers, Senior Reactor Analyst
P. Skinner, DRP, Branch 2 Chief
L. Wheeler, NRR

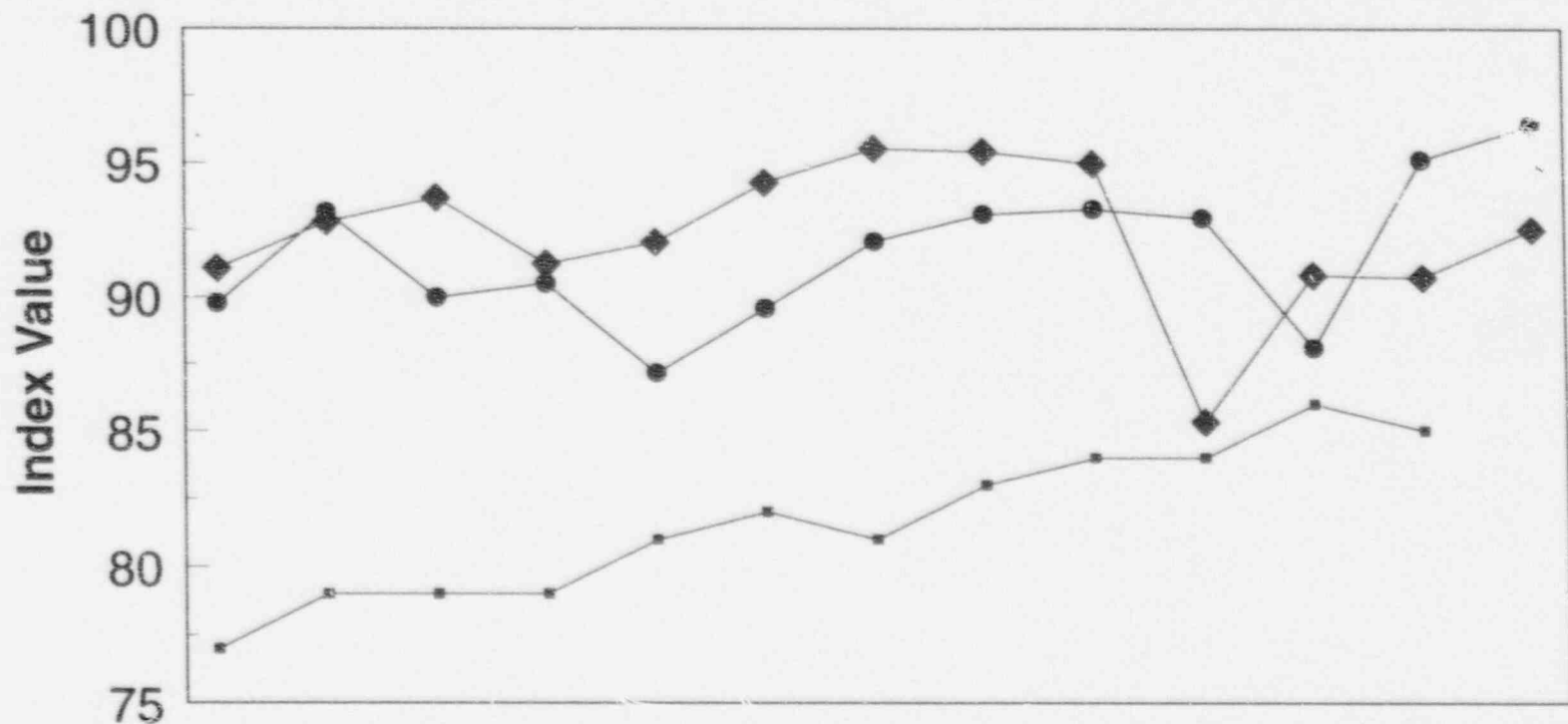
Southern Nuclear Operating Company, Inc.

B. Beasley, Plant Manager
B. Burmiester, Engineering Manager
L. Ward, Engineering & Licensing Manager

Vogtle Electric Generating Plant



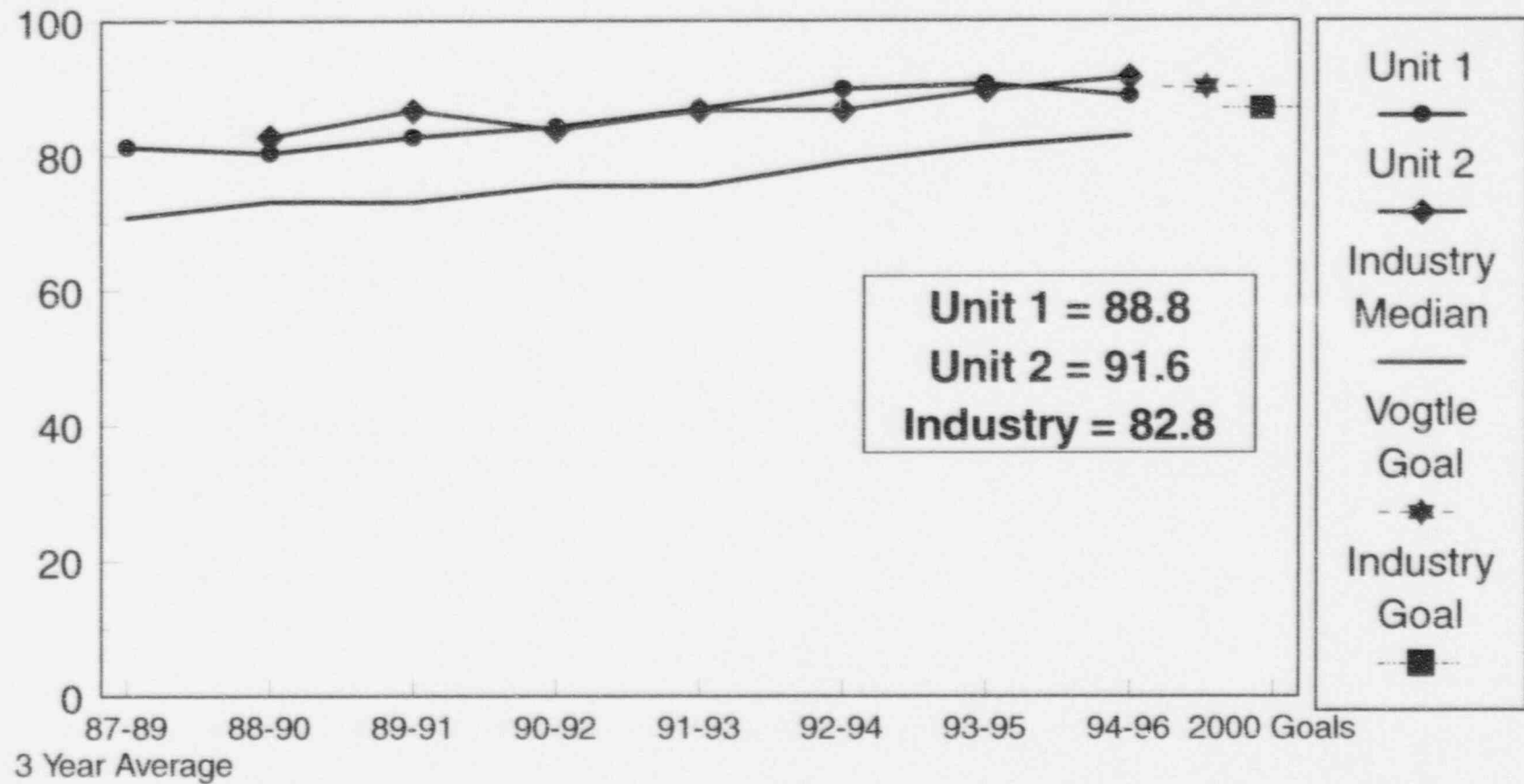
Performance Index



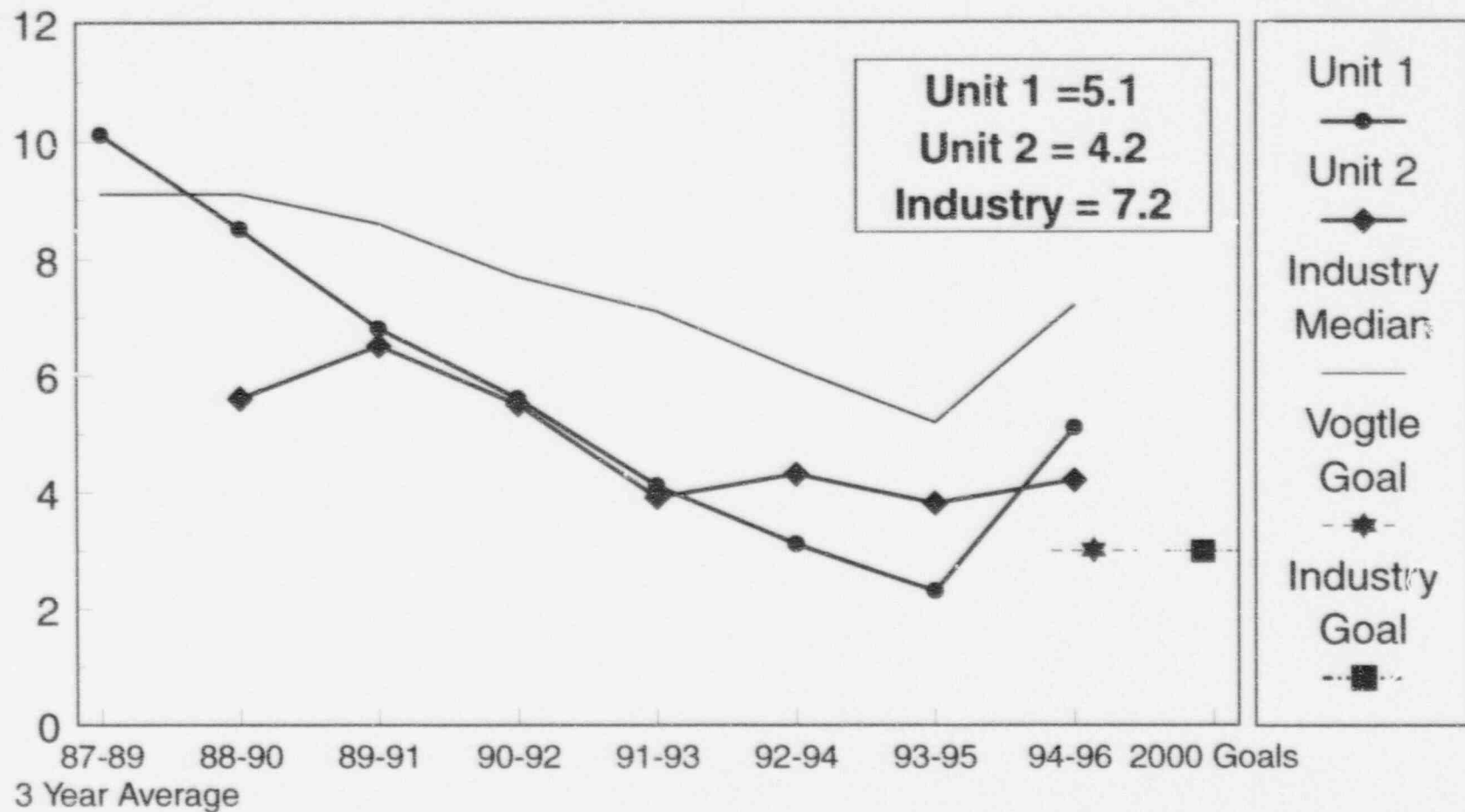
YearQuarter	941	942	943	944	951	952	953	954	961	962	963	964	971
Unit 1	◆ 91.06	92.79	93.69	91.22	92.03	94.25	95.52	95.41	94.96	85.33	90.80	90.70	92.49
Unit 2	● 89.76	93.16	89.98	90.50	87.16	89.57	92.05	93.07	93.26	92.92	88.10	95.10	96.40
Ind. Median	■ 77.00	79.00	79.00	79.00	81.00	82.00	81.00	83.00	84.00	84.00	86.00	85.00	NA

Unit Capability Factor

Percent

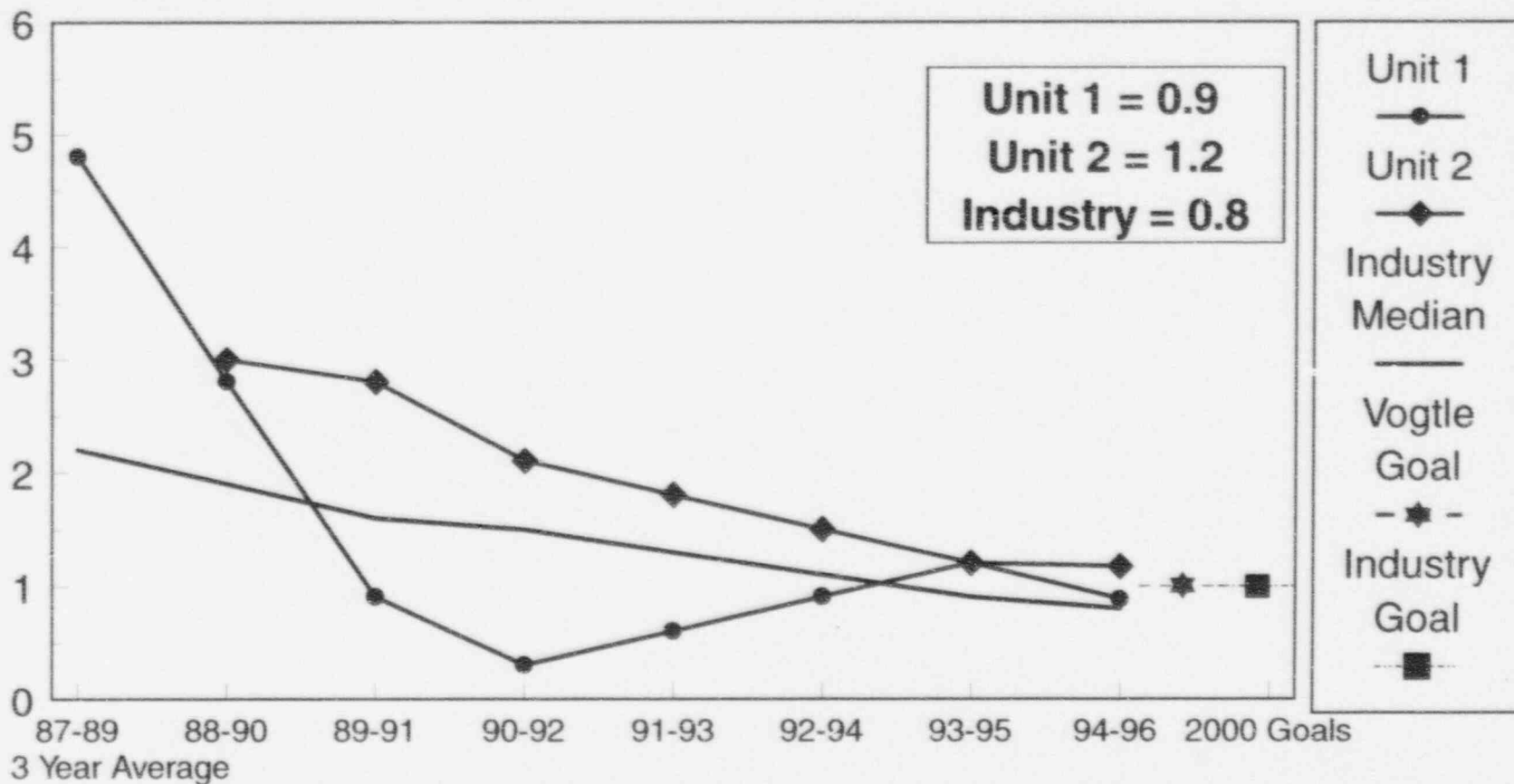


Unplanned Capability Loss Factor



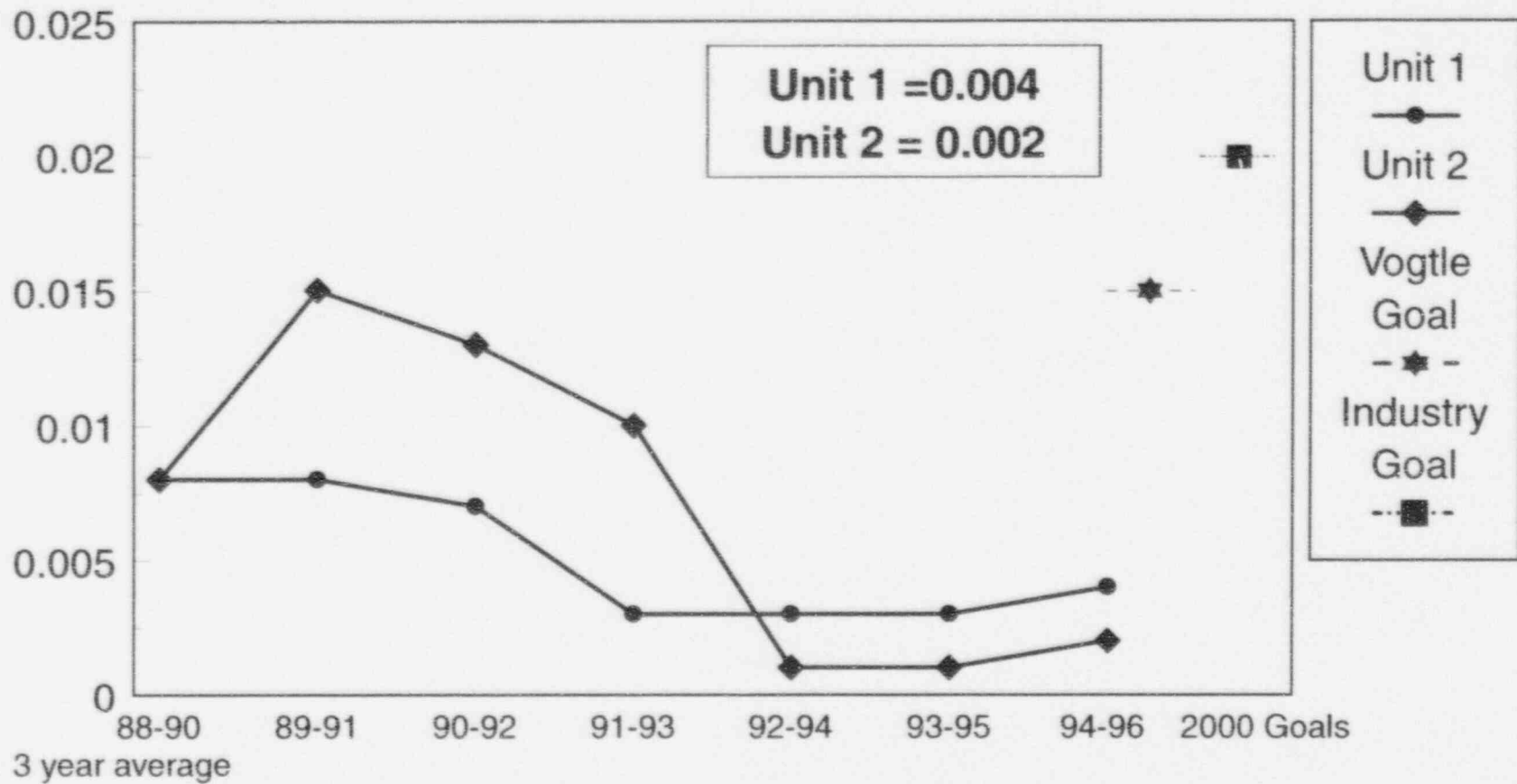
Unplanned Automatic Scrams

Scrams per 7000 Hours Critical



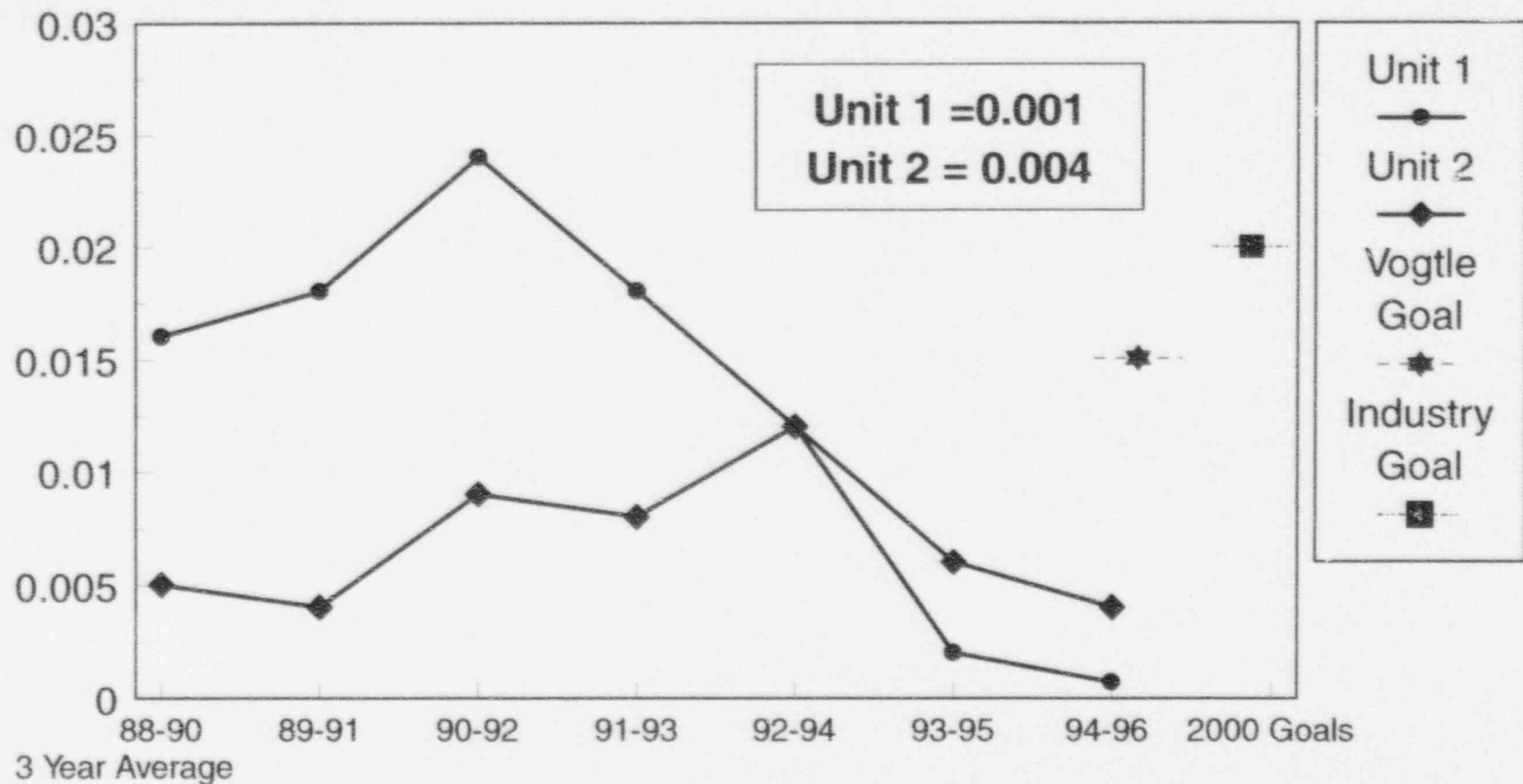
SSPI-High Pressure Safety Injection

Unavailability Factor



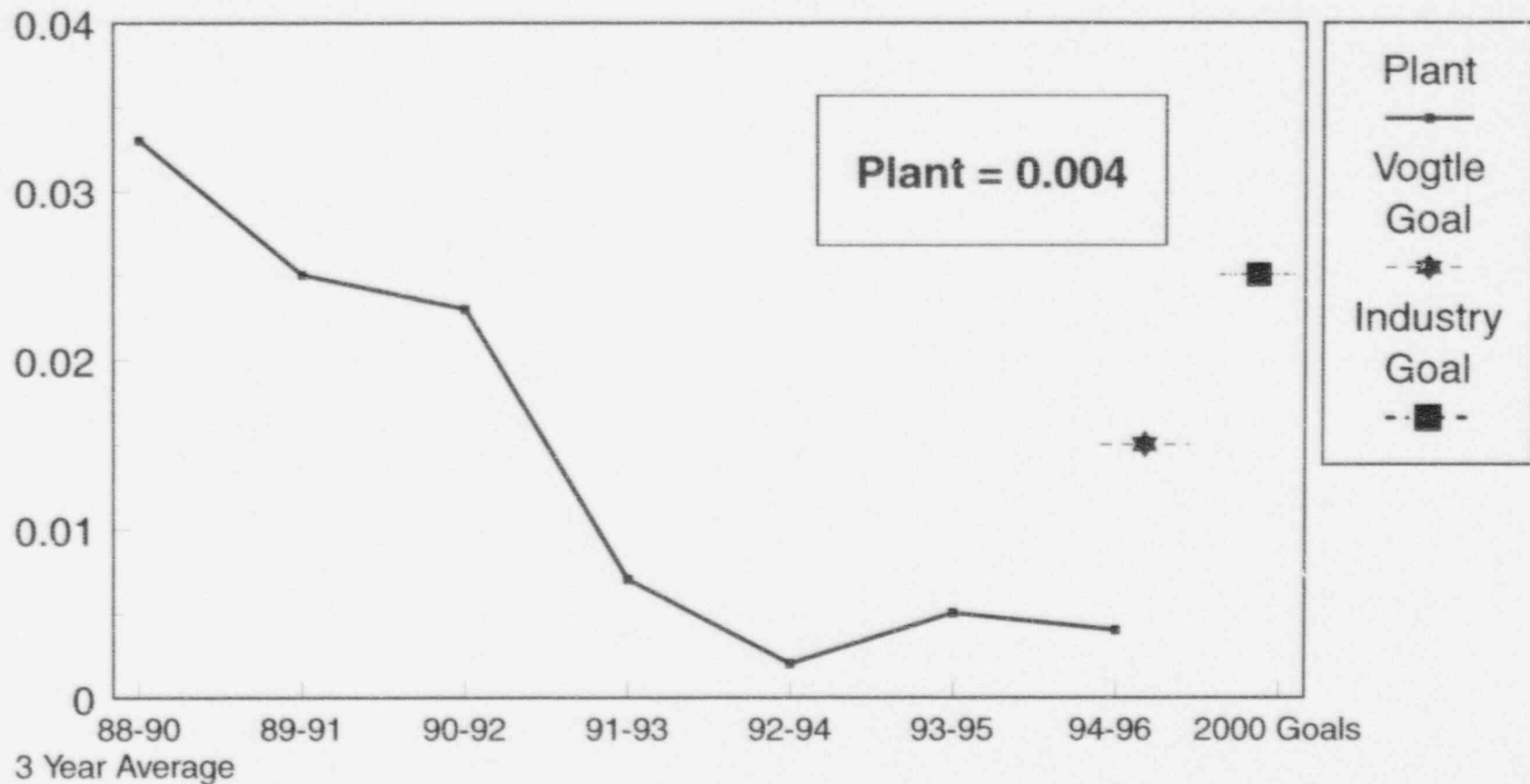
SSPI-Auxiliary Feedwater System

Unavailability Factor



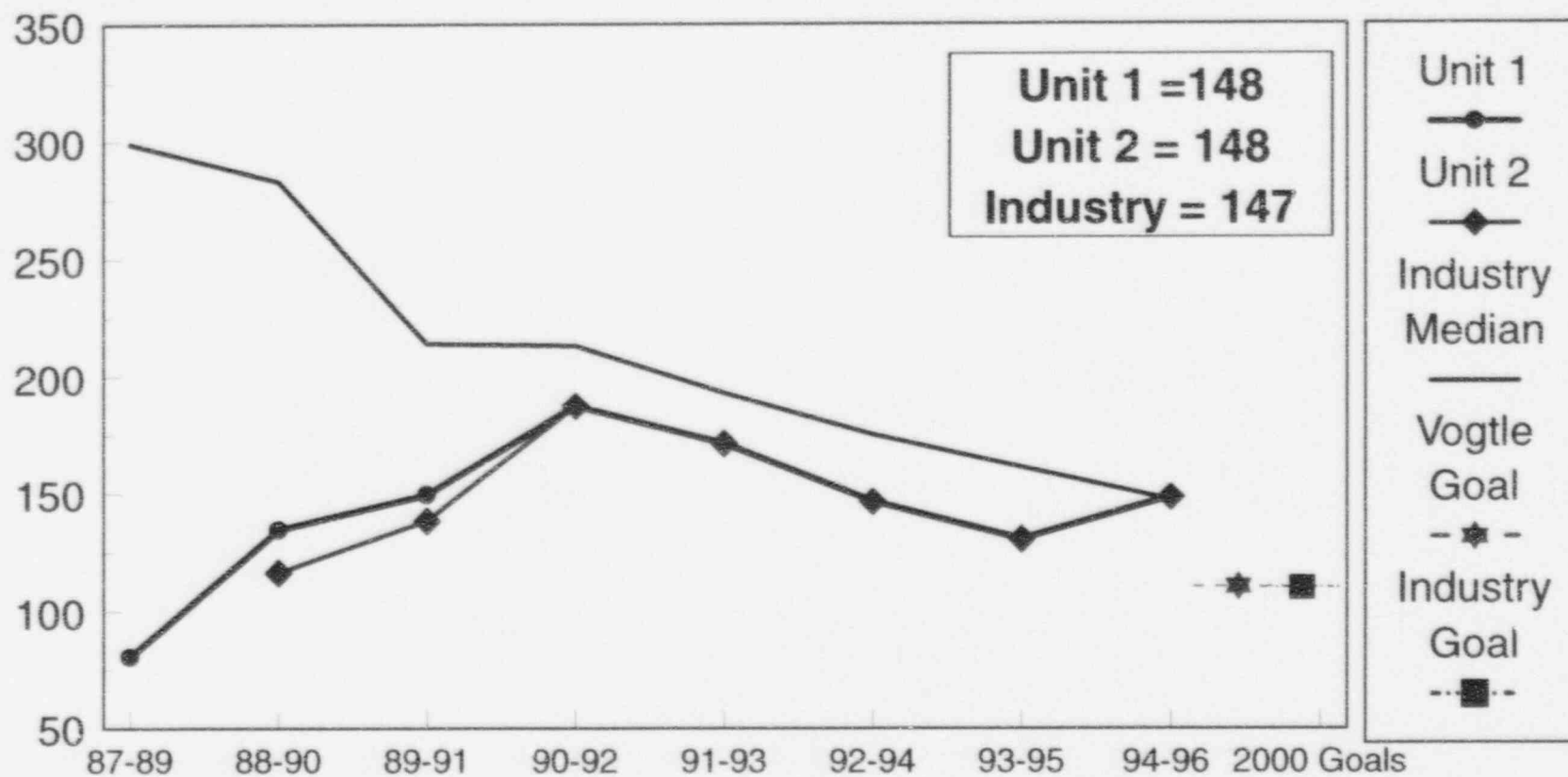
SSPI-Emergency AC Power

Unavailability Factor



Collective Radiation Exposure

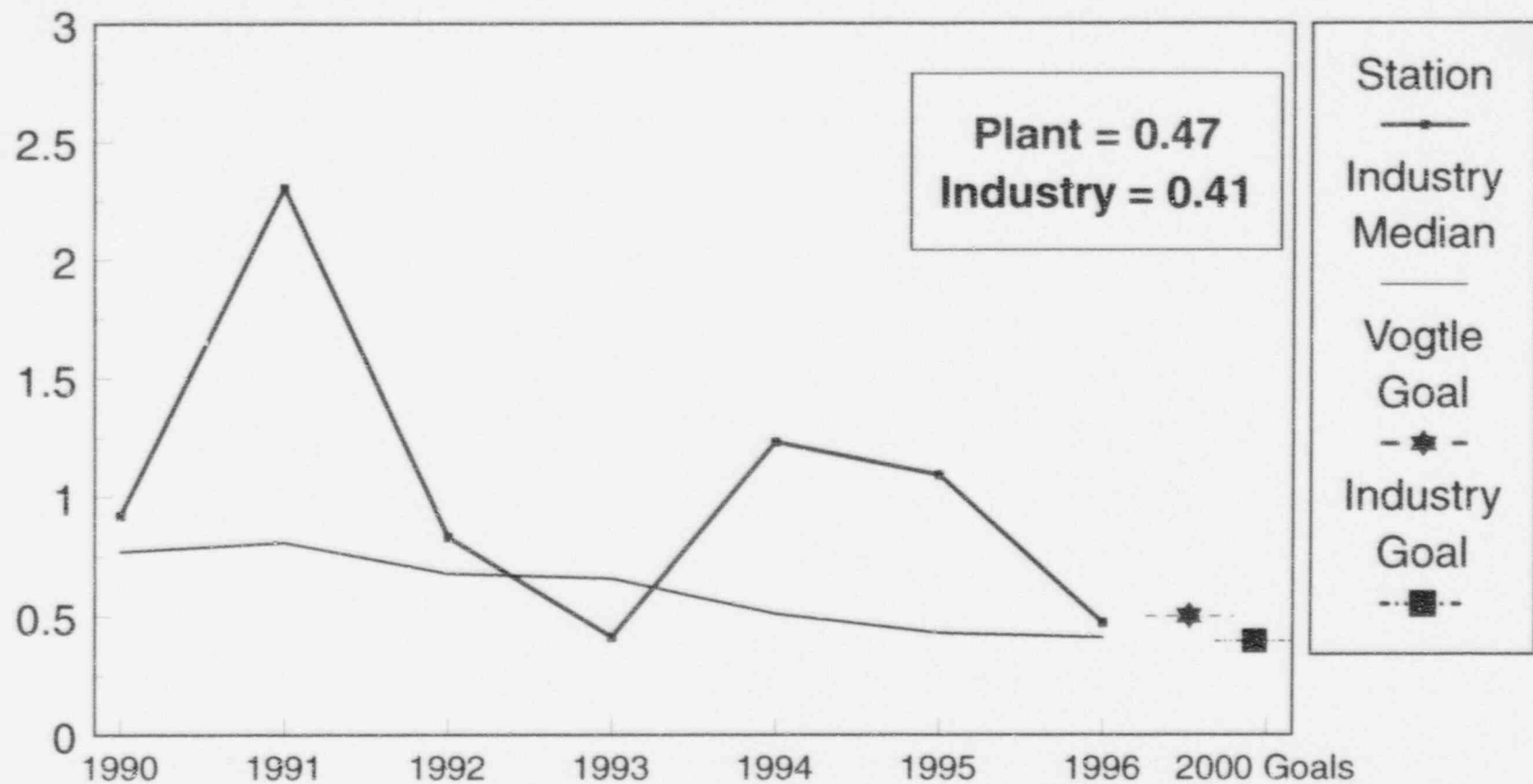
Man-rem



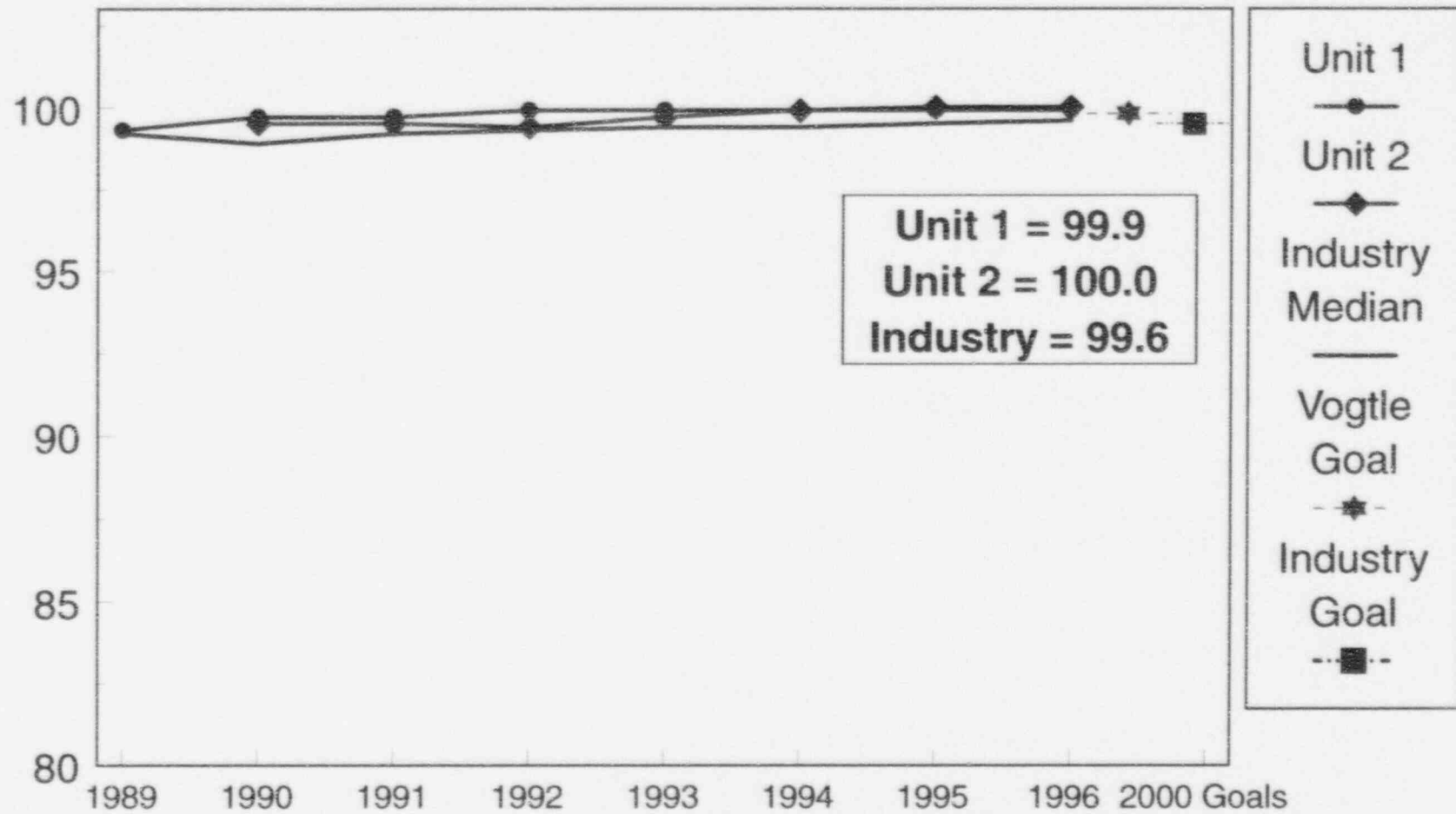
Man-rem per unit per year, 3 year average

Industrial Safety

Accident Rate per 200,000 man-hours

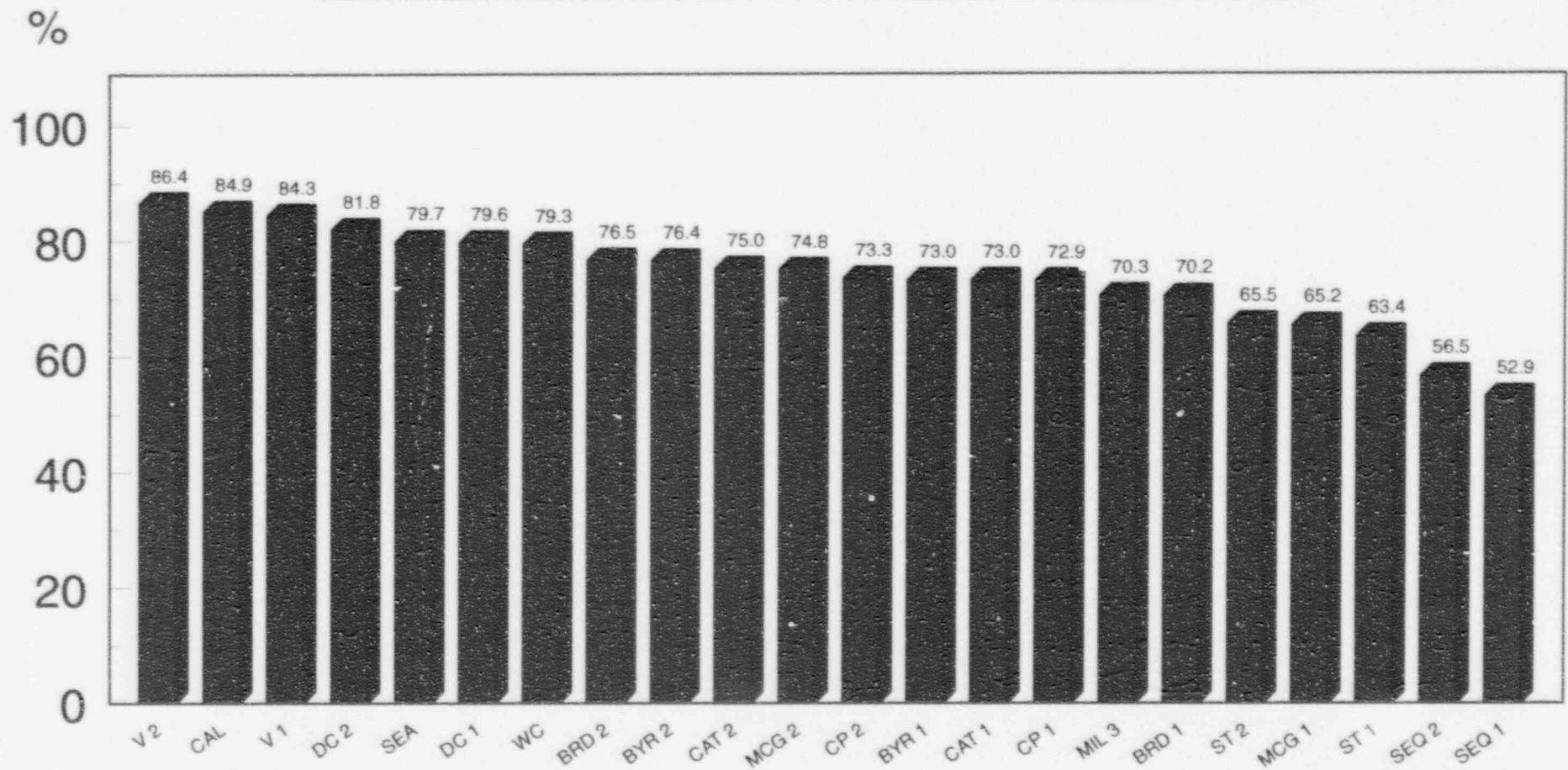


Thermal Performance



PEER PLANT COMPARISON

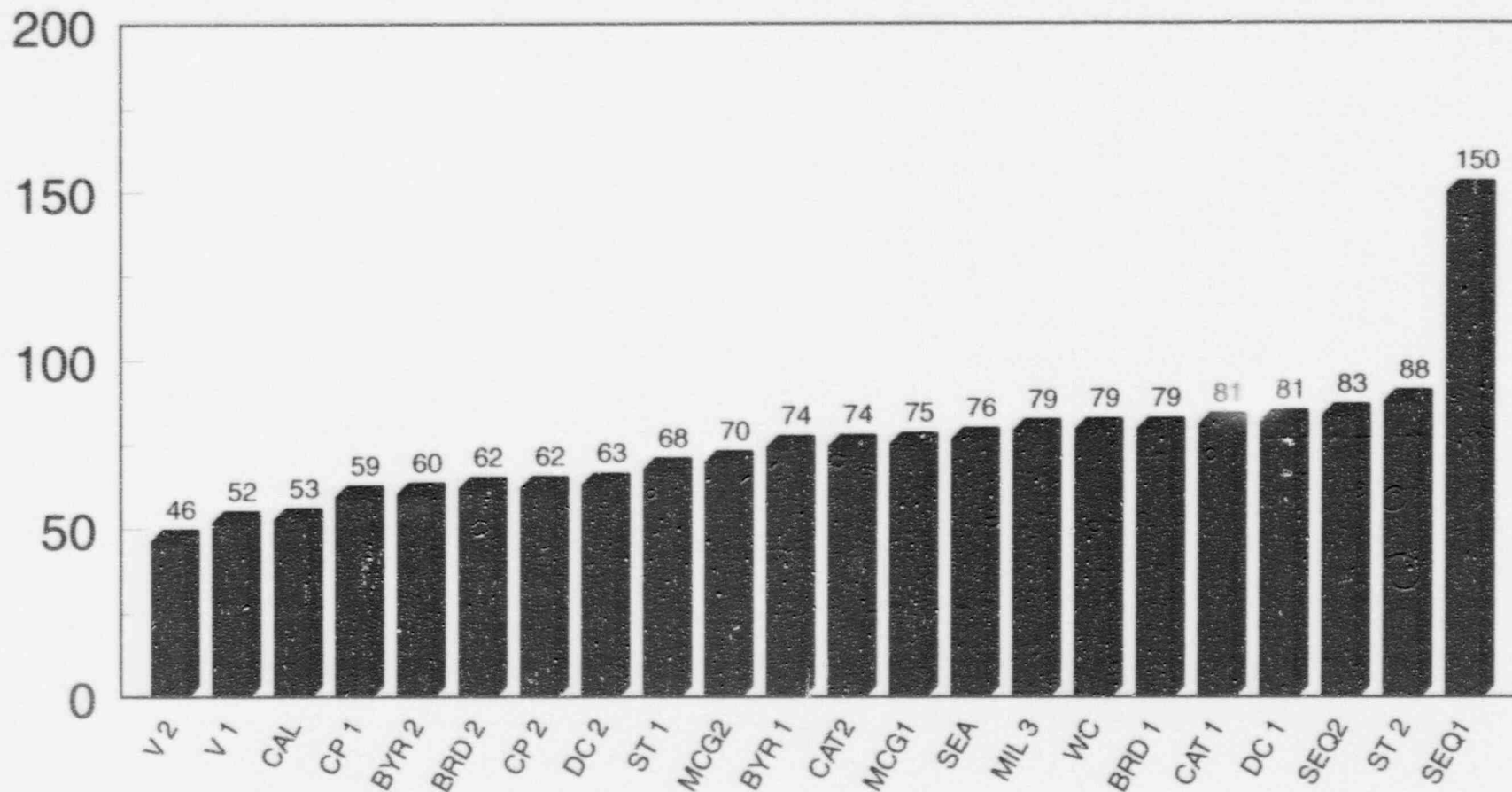
CUMULATIVE CAPACITY FACTOR
DATA THROUGH DECEMBER 1996



DATA PROVIDED BY WESTINGHOUSE NUCLEAR SERVICES

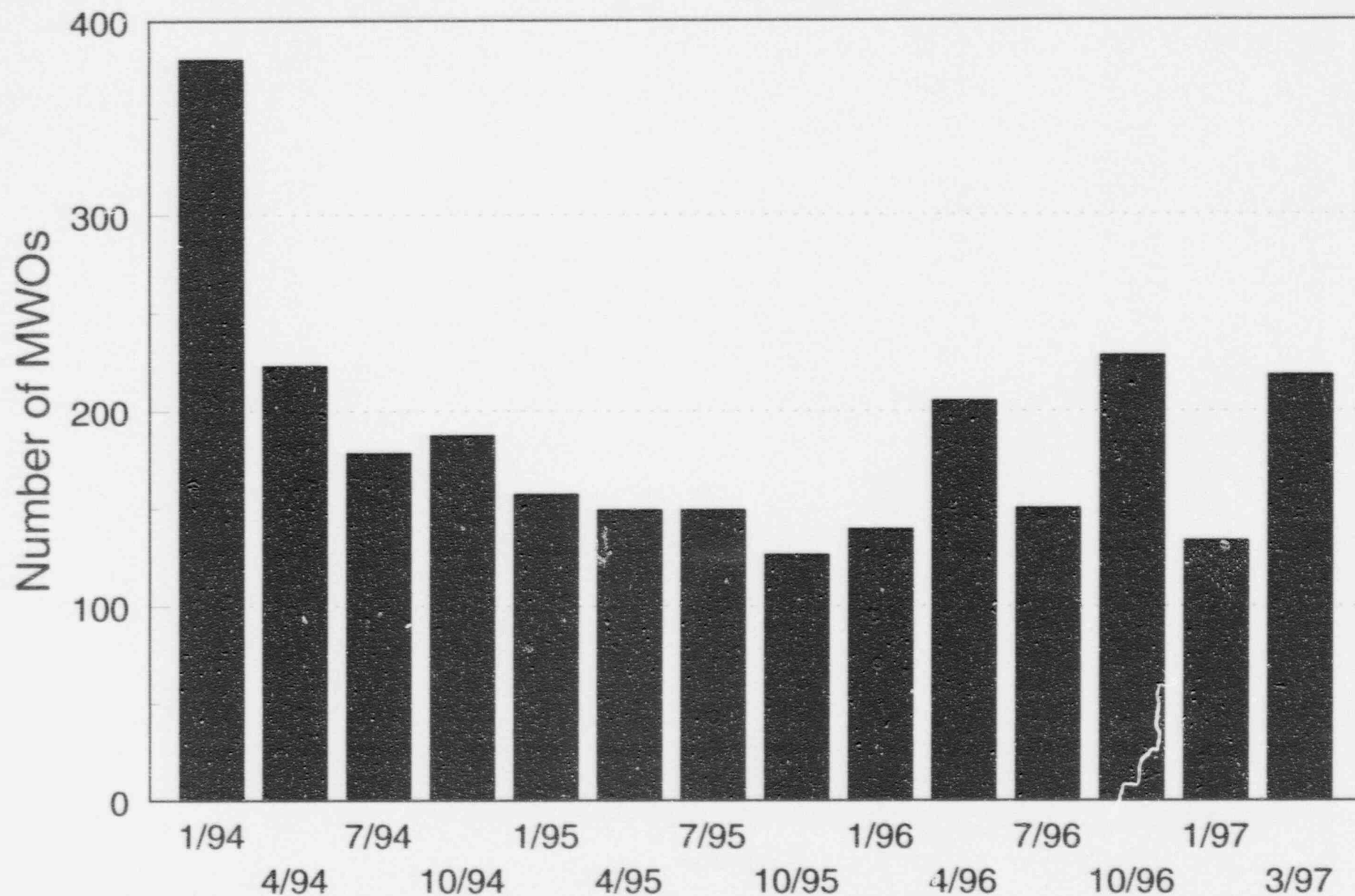
PEER PLANT COMPARISON

AVERAGE OUTAGE LENGTH
DATA THROUGH DECEMBER 1996*

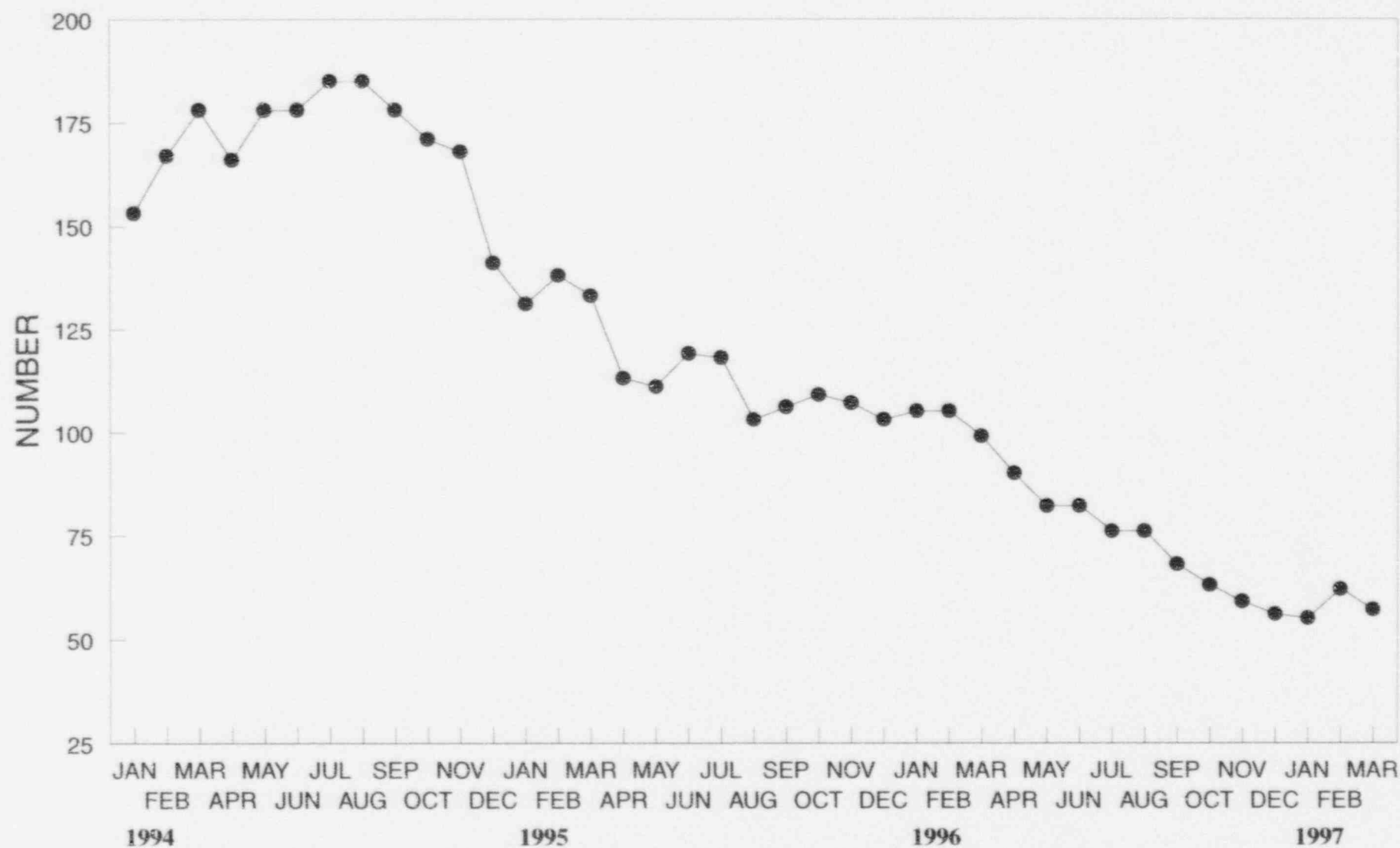


DATA PROVIDED BY WESTINGHOUSE NUCLEAR SERVICES

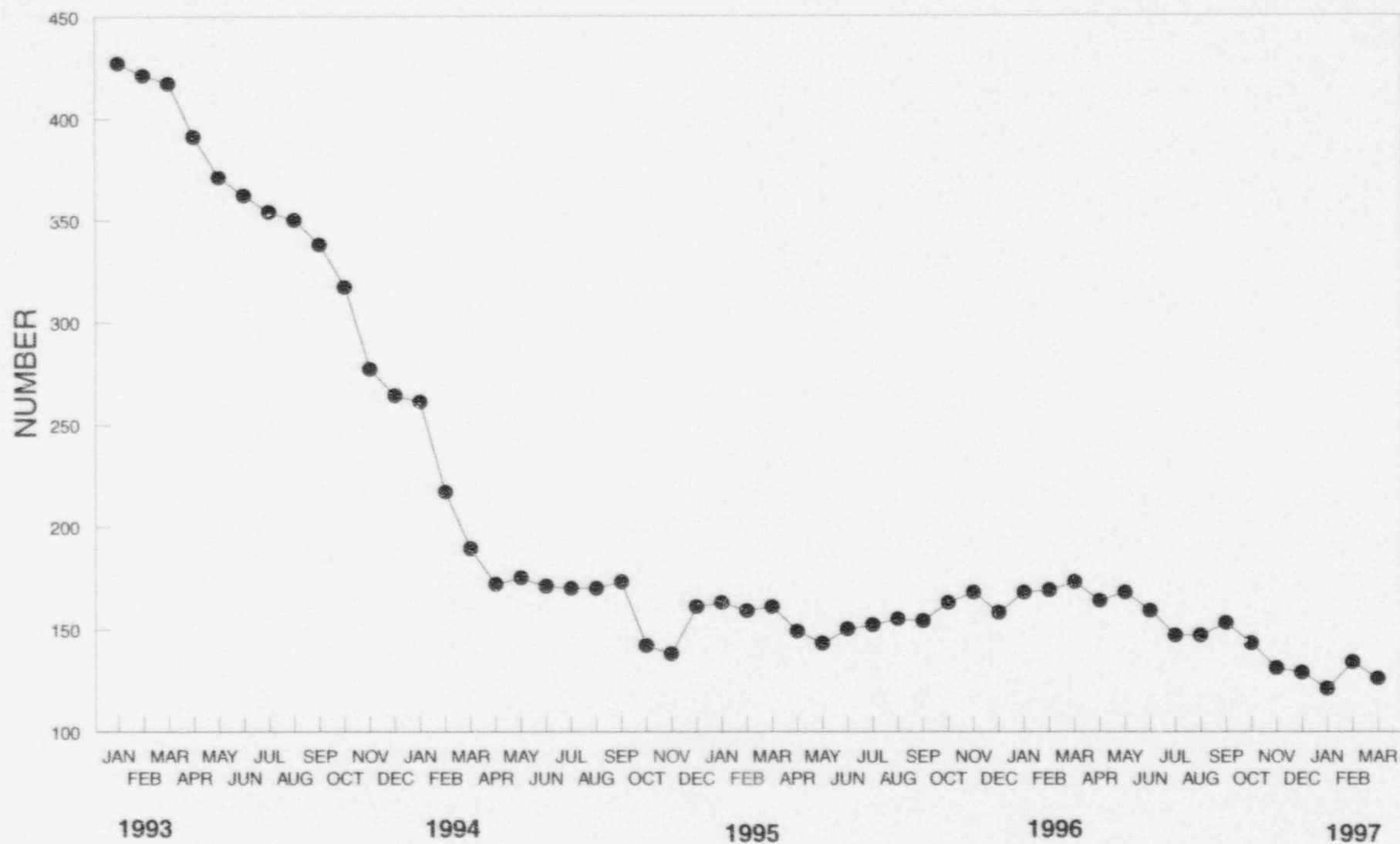
Non-Outage Corrective MWO Backlog Not FWC



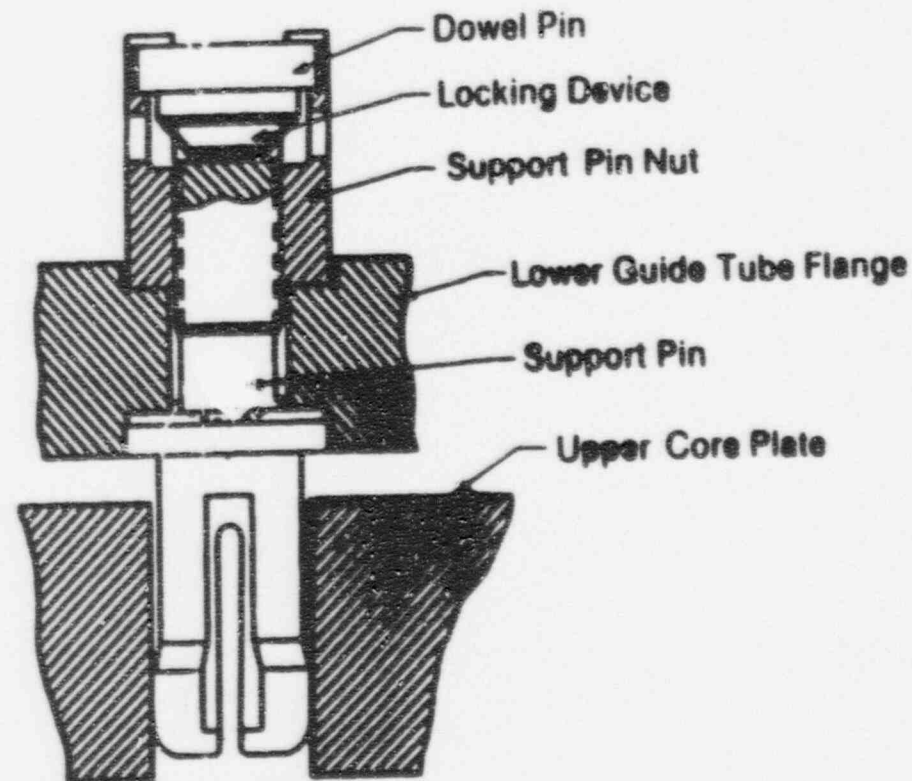
Open MDCs



Open DCRs



Cross Section of Pins with Welded Lock Bar Nut Retainers



Vogtle Electric Generating Plant

Plant Performance Update

April 21, 1997

Major Problems List

- **Control rod guide tube support pin assembly failure.**
- **Steam generator # 4 tube sheet degradation.**
- **Fuel Reliability Issues.**
- **Self Assessment/Complacency.**

Conservative Decision Making

- **During the 1R6 refueling outage, the decision was made not to enter a reduced inventory condition and hence incurred additional critical path outage time.**
- **In May 1996, a manual reactor trip was initiated in response to inadequate feed flow to Unit 1 SG # 4. Rapid identification and response reflected positively.**
- **As a result of the loose part in SG # 4, the controls in place prior to and during fueled midloop were noted to be conservative.**

Conservative Decision Making

- **In October 1996, a manual reactor trip was initiated due to a failed MFRV. The rapid identification and response to the event indicated good performance of the operating crew.**
- **In October 1996, an NOUE was declared due to the loss of the Unit 2 annunciator function. The conservative actions taken in response to this event were considered appropriate.**
- **After a Unit 1 trip due to a maintenance technician touching a metal tag, actions taken by operations were considered conservative.**

NRC Identified Strengths

- **IR-96-02, Strong operator performance during RCS draindown conducted in preparation for reactor vessel head removal.**
- **IR-96-02, The ISEG evaluation of the Unit 1 RCS loss of inventory was noted to be thorough, well done and considered a strength.**
- **IR-96-02, HP postings observed throughout the plant were good and considered a strength.**
- **IR-96-02, The investigative review of the Unit 1 reactor head stud engagement deficiency was considered a strength.**

NRC Identified Strengths

- **IR-96-05, Quick actions to diagnose and initiate manual action to terminate an RCS transient due to a failed open steam dump valve, reflected positively on the operating crew.**
- **IR-96-10, The detailed pre-evolution brief by the performance team for MSRV testing was considered noteworthy.**
- **IR-96-14, The results of an event review team to determine the root cause of failed journal bearing was considered a strength.**

Self Assessment Programs

- **Effective Departmental Assessments are being conducted quarterly. These also serve as a basis for strengthening the training program.**
- **The number of Audit Finding Reports issued by SAER have increased over the last 2 years.**
- **ISEG reviews are effectively used by line management, especially the configuration control assessment and refueling outage risk assessments.**

Self Assessment Programs

- **The threshold for reporting Deficiencies has been lowered.**

1995-357

1996-899

- **Use of industry operating experience.**

Licensee Event Reports

1. Accumulator crosstie.

2. P4 Testing.

Non-Cited Violations

24

Plant Accomplishments

- **VEGP had an 84.2 % capability factor for 1996 and a 98.3 % capability factor for the first quarter of 1997.**
- **2 refueling outages in 1996 with no significant safety problems.**
- **Completed record 411 day operating cycle for Unit 2 prior to the 2R5 refueling outage. 2R5 outage was 34 days and 17 hours.**
- **Successfully completed mid-cycle Unit 1 maintenance outage.**

Plant Accomplishments

- **Maintained high standard of materiel condition for the plant with low work order backlog and focus on the age of the backlog.**
- **Effectively implemented Improved Technical Specifications for both units.**
- **FSAR review completed with no significant safety issues identified. Initiated appropriate action for discrepancies identified.**
- **Operational Safeguards Response Evaluation concluded that overall, the security force demonstrated an effective contingency response capability.**

Plant Accomplishments

- **All the accredited training programs had accreditation renewed. This was accomplished with no INPO identified problems.**
- **Enhanced maintenance training using a flow loop simulator.**
- **Management participation/involvement in training programs.**

Training Programs Re-accreditation

- **Nonlicensed Operator**
- **Reactor Operator**
- **Senior Reactor Operator**
- **Shift Supervisor**
- **Shift Technical Advisor**
- **Instrument and Controls Technician**
- **Electrical Maintenance**
- **Mechanical Maintenance**
- **Chemistry Technicians**
- **Radiological Technicians**
- **Engineering Support Personnel**

Plant Accomplishments

- **Successful Graded Exercise with only one exercise weakness.**
- **Maintenance Performance Team implementation throughout this SALP period.**
- **Health Physics initiated a centralized remote monitoring station.**
- **EDF exchange program.**

Significant Equipment Reliability Improvements

- **The refurbishment of the Nuclear Service Cooling Water Pumps.**
- **The changeout of the Reactor Coolant Pumps.**
- **Partial re-wedge of the Unit 2 Main Generator.**
- **Unit 2 Control rod guide tube support pin assembly inspection. Change-out plans scheduled for the 1R7 and 2R6 refueling outages.**
- **Motor Operator Valve modifications and enhancements.**

Other Plant Challenges

- **Safety Injection motor cooler.**
- **Containment closeout.**
- **Radwaste shipment exceeded limits.**

Engineering & License Initiatives

- Submitted SF rack credit for boron analysis for approval.
- Purchased Maine Yankee boron fuel racks.
- Preparing for 1st SSFA, as stated in 50.54(f) response.
- Analysis of SG #4 tubesheet damage.
- Will provide on-line FSAR, with search capability.
- Conversion of IPE model and development of EOOS for risk evaluation.

OPPORTUNITIES

- DEVELOPMENT OF SYSTEM ENGINEERS
- MAINTENANCE RULE SELF ASSESSMENT /INSPECTION
- SYSTEM PERFORMANCE MONITORING /RELIABILITY INITIATIVES



HIGHLIGHTS

- BI-WEEKLY SYSTEM ENGINEER REFRESHER TRAINING
- MINIMAL ENGINEERING BACKLOG
- AVERAGE PLANT HEAT RATE ~10,000 BTU/KWH
- ALL SSPI AT < 0.5% UN, VAILABILITY
- TRANSITION TO ITS

