

50-338



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

April 30, 1997

Mr. J.P. O'Hanlon  
Senior Vice President - Nuclear  
Virginia Electric and Power Company  
5000 Dominion Blvd.  
Glen Allen, Virginia 23060

SUBJECT: NORTH ANNA UNIT 2 - INSERVICE INSPECTION PROGRAM RELIEF REQUEST  
SPT-16, REVISION 2 (TAC NOS. M94264 AND M94265)

Dear Mr. O'Hanlon:

The NRC staff, with technical assistance from our contractor, the Idaho National Engineering and Environmental Laboratory (INEEL), has reviewed and evaluated the information you provided in letters dated December 7, 1995, June 10, 1996, and February 6, 1997, related to the Second 10-Year Interval Inservice Inspection Program Request for Relief SPT-16 for North Anna Power Station, Unit 2.

In your letter dated December 7, 1995, Virginia Electric and Power Company (VEPCO) submitted a request for authorization to implement Code Cases N-522, *Pressure Testing of Containment Penetration Piping*, and N-535, *Alternative Requirements for Inservice Inspection Intervals*, for North Anna Power Station, Units 1 and 2. However, in your letter dated June 10, 1996, VEPCO withdrew the request to implement Code Case N-535 for both units and Code Case N-522 for Unit 1. VEPCO revised the request for authorization to implement Code Case N-522 for Unit 2 and submitted it as Request for Relief SPT-16. In your letter dated February 6, 1997, VEPCO submitted Revision 2 to Relief Request SPT-16, providing additional clarification.

Based on the information submitted, the staff adopts the contractor's conclusions and recommendations presented in the enclosed Technical Letter Report. The licensee proposes to implement Code Case N-522, *Pressure Testing of Containment Penetration Piping*, supplemented by performing the Appendix J pressure test at the peak calculated containment design pressure and by including a method for the detection and location of leakage. The staff concluded that the pressure-retaining integrity of the containment isolation valves and connecting piping and their associated safety functions will be verified with an Appendix J, Type C test if it is conducted at the peak calculated containment pressure as committed by the licensee. Therefore, the licensee's proposed alternative in Request for Relief SPT-16 is authorized pursuant to 10 CFR 50.55a(a)(3)(i) as providing an acceptable level of quality and safety.

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This completes our efforts on this issue and we are, therefore, closing out TAC Nos. M94264 (Unit 1) and M94265 (Unit 2).

Sincerely,

(Original Signed By)

Mark Reinhart, Acting Director  
Project Directorate II-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-339

Enclosures:

1. Safety Evaluation
2. Technical Letter Report

cc w/enclosures: See next page

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COPY	Yes/No	Yes/No	Yes/No	Yes/No	Yes/No	Yes/No

Mr. J. P. O'Hanlon  
Virginia Electric & Power Company

North Anna Power Station  
Units 1 and 2

cc:

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