

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)										DOCKET NUMBER (2)				PAGE (3)	
Browns Ferry - Unit 2										0 5 0 0 0 2 6 1 0				1 OF 0 2	

TITLE (4)

Unqualified Reactor Protection System Instrument Panels

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
									Browns Ferry - Unit 1	0 5 0 0 0 2 5 9
1	2	20	8	5	8	5	0	2	0	0
									Browns Ferry - Unit 3	0 5 0 0 0 2 9 6

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)				
N		20.402(b)	20.406(c)		50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)	01010	20.406(a)(1)(i)	50.36(c)(1)	X	50.73(a)(2)(v)	73.71(c)
		20.406(a)(1)(ii)	50.36(c)(2)		50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.406(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(vii)(A)	
		20.406(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(vii)(B)	
		20.406(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER	
	AREA CODE	
Stephen B. Jones, Compliance Engineer	2 0 5	7 2 9 - 2 5 3 8

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single space typewritten lines) (16)

A design analysis of eight unit 2 reactor protection system load instrument panels identified as not being constructed per design drawings discovered two of the panels were not fully seismically qualified. The panel configuration is being changed because of the environmental qualification program, and the new configuration will be seismically qualified. In the present plant configuration, the instruments located on these panels are not required by TS.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Browns Ferry - Unit 2	0500026085	-	020	-	010	02	OF 02

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Unit 1 and unit 2 were in refueling outages. Unit 3 was in an extended maintenance outage. Only unit 2 was affected by this event.

On June 27, 1985, eight local reactor protection system (RPS) panels on each unit were identified as not being constructed per design drawing. The as-constructed configuration of the panels identified on unit 3 was found to be seismically qualified in an August 1985 engineering analysis. The analysis of the unit 2 RPS panels was transmitted to the site design branch on November 18, 1985, and plant management was notified on December 20, 1985. This analysis determined that six of the panels were seismically qualified but the remaining panels, 25-5-001 and 25-6-001, could not be qualified. The analysis showed that the panel anchors would see high loads during a seismic event; and because the anchor bolt material could not be identified, it was conservatively determined a failure would occur. In the present plant configuration, technical specifications (TS) do not require the instrumentation to be operable; therefore, no immediate actions were required.

This condition was the result of the actual constructed configuration never being properly documented during construction. Therefore, an accurate analysis of the panel was not possible.

The instruments on these panels are being replaced as part of the Environmental Qualification Program. The new instruments will be placed on new instrument panels. The seismic qualification of the new panels will be verified and panels 25-5-001 and 25-6-001 analyzed for their new configuration and, if necessary, these panels will be modified prior to restart of unit 2. The unit 1 RPS panels will be analyzed and corrective action completed prior to restart of unit 1. The configuration control program currently in place identified this problem and should preclude recurrence of this type problem on future installations.

The instruments located on these panels monitor reactor water level and reactor pressure, initiate scrams, group 2, 3, and 6 isolations, and standby gas treatment (BH). The high pressure coolant injection (BJ) and reactor core isolation cooling (BN) turbines are tripped by a high water level signal from these instruments. In the present unit 2 configuration, these instruments are not required by TS; therefore, there is no immediate effect on the safety of the plant.

Responsible Plant Section - NA

Previous Events - None

TENNESSEE VALLEY AUTHORITY

Browns Ferry Nuclear Plant
P.O. Box 2000
Decatur, Alabama 35602

January 17, 1986

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 - DOCKET
NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE OCCURRENCE
REPORT BFRO-50-260/85020

The enclosed report provides details concerning the unqualified reactor
protection system instrument panels. This report is submitted in
accordance to 10 CFR 50.73 (a)(2)(v).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

Robert L. Lewis

Robert L. Lewis
Plant Manager
Browns Ferry Nuclear Plant

Enclosures

cc (Enclosures):

Regional Administrator
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

INPO Records Center
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Resident Inspector, Browns Ferry Nuclear Plant

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