

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 8 7				PAGE 15 1 OF 0 2	
TITLE (4) Unplanned Engineered Safety Feature Actuation When Wrong Breaker Opened.															
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)		
0	2	1	6	8	6	0	0	3	SSES - Unit 2				0 5 0 0 0 3 8 8		
0	2	1	6	8	6	0	0	3	0 3 1 4 8 6				0 5 0 0 0		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)													
4		20.402(b)				20.400(e)				<input checked="" type="checkbox"/> 80.73(a)(2)(iv)				73.71(b)	
POWER LEVEL (10)		20.406(a)(1)(i)				80.36(e)(1)				80.73(a)(2)(v)				73.71(e)	
0 10 10		20.406(a)(1)(ii)				80.36(e)(2)				80.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
		20.406(a)(1)(iii)				80.73(a)(2)(i)				80.73(a)(2)(vii)(A)					
		20.406(a)(1)(iv)				80.73(a)(2)(ii)				80.73(a)(2)(vii)(B)					
		20.406(a)(1)(v)				80.73(a)(2)(iii)				80.73(a)(2)(x)					
LICENSEE CONTACT FOR THIS LER (12)															
NAME T.N. Creasy										TELEPHONE NUMBER					
										AREA CODE 7 1 7 5 4 2 - 3 2 4 2					
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)															
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS					
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR	
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On February 16, 1986 at 0900 a partial loss of power occurred to the Division I Nuclear Steam Supply Shutoff System. While applying protective blocking to a breaker on the Reactor Protection System (RPS), an operator opened circuit breaker CB3A as specified on the blocking permit. However, the breaker was incorrectly labeled and was actually RPS circuit breaker CB5A. This caused a loss of Shutdown Cooling, isolation of the Reactor Water Cleanup System, isolation of the Zone III Heating, Ventilation and Air Conditioning System, initiation of the 'A' Standby Gas Treatment System and 'A' Control Room Emergency Outside Air Supply System, and various other inboard valve isolations. The breaker was reclosed, the isolation reset, and shutdown cooling restored by 0920.

Four out of eight breakers on both Unit 1 and Unit 2 'A' RPS buses were found to be mislabeled. Temporary labels correctly identifying the circuit breakers were installed the same day. Permanent labels have since been installed. Other corrective actions are still under review pending the results of a Human Performance Evaluation System (HPES) investigation.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Susquehanna Steam Electric Station Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 8 7 8 6	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		— 0 0 3	— 0 0	0 3	OF	0 3	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On February 16, 1986 at 0900 a partial loss of power occurred to the Division I Nuclear Steam Supply Shutoff System (NSSSS, EIIS Code: JM). An operator (utility, non-licensed) was applying protective blocking to a breaker on the Reactor Protection System (RPS, EIIS Code: JC) to support maintenance activities. The operator located and opened RPS circuit breaker CB3A as specified on the blocking permit, however, the breaker was incorrectly labeled and was actually RPS circuit breaker CB5A. This caused a loss of shutdown cooling due to isolation of the inboard suction valve, isolation of the Reactor Water Cleanup System (EIIS Code: CE), isolation of the Zone III Heating, Ventilation, and Air Conditioning System (EIIS Code: VA), initiation of the 'A' Standby Gas Treatment System and 'A' Control Room Emergency Outside Air Supply System (EIIS Code: RH), and various other inboard valve isolations. The operator reclosed breaker CB5A (labeled CB3A) and the isolation logic was reset at 0905. Shutdown cooling was restored at 0920, and the affected systems were restored as applicable.

Subsequent investigation revealed four out of eight breakers on the 'A' RPS bus were incorrectly labeled. All circuit breakers on the 'B' RPS bus were labeled correctly. A similar situation existed on Unit 2, in that four of eight breakers on the 'A' RPS bus were labeled incorrectly while the 'B' RPS bus labeling was correct. The breakers had been mislabeled since June 13, 1985.

Temporary labels correctly identifying the circuit breakers were installed on the Unit 1 and Unit 2 'A' RPS buses the same day. Permanent labels have since been installed. Other corrective actions are still under review pending the results of a Human Performance Evaluation System (HPES) investigation.



Pennsylvania Power & Light Company

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March 14, 1986

U.S. Nuclear Regulatory Commission
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Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSE EVENT REPORT 86-003-00
ER 100450 FILE 841-23
PLAS- 151

Docket No. 50-387
License No. NPF-14

Attached is Licensee Event Report 86-003-00. This event was determined reportable per 10CFR50.73(a)(2)(iv), in that an unplanned Engineered Safety Feature (ESF) actuation occurred.

T.M. Crimmins, Jr.
Superintendent of Plant-Susquehanna

TNC/pjg

cc: Dr. Thomas E. Murley
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