

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Wolf Creek Generating Station										DOCKET NUMBER (2) 0 5 0 0 0 4 8 2										PAGE (3) 1 OF 0 2	
TITLE (4) Technical Specification Violation - Fire Dampers Not Installed																					
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)									
1	2	0	8	5	8	5	0	8	6	0	0	1	1	6	8	6	0	5	0	0	0
OPERATING MODE (9) 1		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)																			
POWER LEVEL (10) 11010		20.402(b)				20.406(a)				80.73(a)(2)(iv)				73.71(b)							
		20.406(a)(1)(i)				80.36(a)(1)				80.73(a)(2)(v)				73.71(a)							
		20.406(a)(1)(ii)				80.36(a)(2)				80.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
		20.406(a)(1)(iii)				80.73(a)(2)(i)				80.73(a)(2)(vii)(A)											
		20.406(a)(1)(iv)				80.73(a)(2)(ii)				80.73(a)(2)(vii)(B)											
		20.406(a)(1)(v)				80.73(a)(2)(iii)				80.73(a)(2)(ix)											
LICENSEE CONTACT FOR THIS LER (12)												TELEPHONE NUMBER									
NAME Merlin G. Williams - Superintendent of Regulatory, Quality and Administrative Services												AREA CODE 3 1 1 6 3 1 6 4 1 1 8 1 8 1 3 1									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC											
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR					
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO									

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On December 20, 1985, at approximately 1430 CST, it was discovered that a violation of Technical Specification 3.7.11 existed as a result of the required fire dampers not being installed in two ventilation penetrations through three hour fire walls in the Auxiliary Building. Technical Specification 3.7.11 requires, in part, that all sealing devices (fire dampers) in fire rated assemblies be operable at all times.

Through a design oversight, the installation of these fire dampers was not required by the design drawings. This situation has been in existence since receipt of the facility license (March 11, 1985). During this time, plant conditions have varied from Mode 6, Refueling, through Mode 1, Power Operation, at reactor power levels of up to 100 percent.

Upon discovery of this condition, an hourly fire watch patrol of the affected area was established in accordance with Action Statement "a" of Specification 3.7.11. A design change to install the required fire dampers is being prepared.

There was no damage to plant equipment or release of radioactivity as a result of this event, and at no time did conditions develop that may have posed a threat to the health or safety of the public.

Licensee Event Reports 85-78-01 and 85-083-00 discuss similar previous occurrences.

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PDR ADOCK 05000482
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Wolf Creek Generating Station	0 5 0 0 0 4 8 2	8 5	- 0 8 6	- 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On December 20, 1985, at approximately 1430 CST, it was discovered that a violation of Technical Specification 3.7.11 existed as a result of fire dampers [VF-BDMP] not being installed in two ventilation penetrations through three hour fire walls in room 1405 of the Auxiliary Building [NF]. Technical Specification 3.7.11 requires, in part, that all sealing devices (fire dampers) in fire-rated assembly penetrations shall be operable at all times.

Wolf Creek was notified of the potential for the existence of this situation by the Architect Engineer in response to a similar discovery at another plant. A field inspection of the affected penetration assemblies revealed that the required fire dampers were not installed in the two penetrations. Upon discovery of this situation, an hourly fire watch patrol of the area was established in accordance with Action Statement 'a' of Technical Specification 3.7.11.

Through a design oversight, the installation of these fire dampers had not been required by the design drawings. Since this situation has been in existence since receipt of the facility license (March 11, 1985), this Technical Specification violation has been in existence since that time. During this time, plant conditions have varied from Mode 6, Refueling, through Mode 1, Power Operation, at reactor power levels of up to 100 percent power.

Room 1405, located on the 2026 foot elevation of the Auxiliary Building, contains the Data Acquisition System for Steam Generator "D" and the resin loading chute for the demineralizers located on the 2000 foot elevation. The need to designate this room as a separate fire area was determined late in the design of the plant. It is believed that a cognitive personnel error led to the failure to include fire dampers in the two ventilation penetrations, although specific details are unknown.

A design change is being developed to require the installation of fire dampers in the two ventilation penetrations. A review of all other three hour fire barrier ventilation penetrations has been conducted to ensure that the proper sealing assemblies have been included in the design drawings.

Fire detection equipment which annunciates in the Control Room is provided in Room 1405 and the immediately adjacent areas. The annunciation in the Control Room ensures prompt response in the event of a fire in the area which would minimize plant equipment damage.

There was no release of radioactivity as a result of this situation, and at no time was there a threat to the health or safety of the public. Licensee Event Reports 85-078-01 and 85-083-00 discuss similar situations which resulted from design drawing omissions.



KANSAS GAS AND ELECTRIC COMPANY

GLENN L. KOESTER
VICE PRESIDENT - NUCLEAR

January 16, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

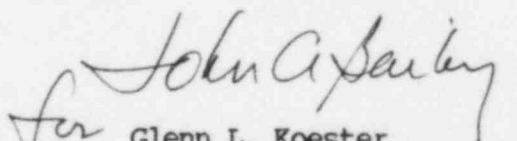
Mr. E.H. Johnson, Acting Director
Division of Reactor Safety and Projects
U.S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

KMLNRC 86-009
Re: Docket No. STN 50-482
Subj: Licensee Event Report 85-086-00

Gentlemen:

The attached Licensee Event Report is submitted pursuant to 10 CFR 50.73 (a) (2) (i) concerning a Technical Specification violation.

Yours very truly,


for Glenn L. Koester
Vice President - Nuclear

GLK:see

Enclosure

xc: PO'Connor (2), w/a
JCummins, w/a

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