



Northern States Power Company

414 Nicollet Mall  
Minneapolis, Minnesota 55401  
Telephone (612) 330-5500

January 13, 1986

Director  
Office of Nuclear Reactor Regulation  
U S Nuclear Regulatory Commission  
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT  
Docket Nos. 50-282 License Nos. DPR-42  
50-306 DPR-60

License Amendment Request dated January 13, 1986  
Change to Westinghouse Fuel and Miscellaneous Changes

Reference: Meeting Minutes prepared by the NRC dated July 31, 1985  
entitled, "Summary of June 26, 1985 Meeting Associated with  
Upcoming Changes to the Reactor Internals and Impact on the  
Technical Specifications"

Attached are three originals and 37 conformed copies of a request for  
change to the Technical Specifications, Appendix A, of Operating Licenses  
DPR-42 and DPR-60. In accordance with 10 CFR Part 170, a check in the  
amount of \$150.00 is enclosed.

This change proposes the Technical Specification changes necessary for  
the operation of Unit 1 Cycle 11. A change in fuel suppliers has been made  
for Unit 1 Cycle 11. Westinghouse will supply fuel for Unit 1 Cycle 11  
and subsequent cycles. Two hardware changes will be incorporated  
in Unit 1 Cycle 11. The upper internals are being replaced with new  
internals of the "inverted top hat" design. Also, thimble plugs will be  
removed from fuel assemblies for the first time for Unit 1 Cycle 11. A  
safety evaluation of the change to the new upper internals will be  
submitted for your information in the near future. A safety evaluation for  
the thimble plug removal has been submitted and approved by the NRC Staff  
in an SER dated October 18, 1985. These hardware changes do not affect the  
Technical Specifications, but have been accounted for in the supporting  
analyses, where appropriate.

Changes incorporated in this submittal are: 1) referencing the WRB-1 DNB  
correlation, 2) temperature coefficient changes, 3) accumulator  
specification changes, 4) peaking factor changes, and 5) deletion of the  
third line segment of the K(z) curve.

Exhibit A describes the proposed changes, provides the reasons for the

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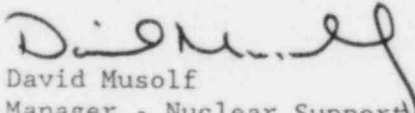
Dir, NRR  
January 13, 1986  
Page 2

changes, and provides the supporting significant hazards evaluation. Exhibit B contains the proposed changes marked up on the existing Technical Specification pages. Exhibit C contains the revised Technical Specification pages.

The information contained in Exhibit D was requested by the NRC Staff at the referenced June 26, 1985 meeting discussing this submittal. Exhibits E thru H support the evaluations in Exhibit A.

In addition to supporting this amendment, Exhibit F and G, the small break LOCA analyses for Westinghouse and Exxon fuel, are being submitted to satisfy the requirements of NUREG 0737, Item II.K.3.31.

Please contact us if you have any questions concerning this License Amendment Request. NRC Staff review and approval of these changes is requested prior to April 15, 1986 to support the operation of Unit 1 Cycle 11.

  
David Musolf  
Manager - Nuclear Support Services

DMM/TMP/tp

c: Regional Administrator-III, NRC  
NRR Project Manager, NRC  
Resident Inspector, NRC  
MPCA  
Attn: J W Ferman  
G Charnoff

Attachments:

Affidavit

- Exhibit A - Evaluation of Proposed Changes to the Technical Specifications
- Exhibit B - Proposed Changes Marked Up on Existing Technical Specification Pages
- Exhibit C - Revised Technical Specification Pages
- Exhibit D - Prairie Island Unit 1 Westinghouse OFA Transition Reloads
- Exhibit E - Demonstration of the Conformance of the Prairie Island Units to Appendix K and 10CFR50.46 for Large Break LOCAs
- Exhibit F - Demonstration of the Conformance of the Prairie Island Units to Appendix K and 10CFR50.46 for Small Break LOCAs
- Exhibit G - Demonstration of the Conformance of the Prairie Island Units with Exxon Fuel Assemblies to Appendix K and 10CFR50.46 for Small Break LOCAs
- Exhibit H - Safety Evaluation of Increased FQ, FdH and Isothermal Temperature Coefficient