

ATTACHMENT A

PROPOSED CHANGES TO APPENDIX A
TECHNICAL SPECIFICATIONS OF
FACILITY OPERATING LICENSE NPF-37

Revised Pages: 3/4 3-15
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
ATTACHMENT B

REASONS FOR PROPOSED CHANGES

- 1) On Table 3.3-3 (pg. 3/4 3-15) item 1d, "(Above P-7)" is changed to "(Above P-11)". This is a typographical error since the ESF function of a safety injection on low pressurizer pressure is enabled above P-11 and not P-7.
- 2) In surveillance 4.6.1.5 (pg 3/4 6-7) delete the periods after locations A, B, C and D. This change is typographical.
- 3) On Table 3.7-5b (pg 3/4 7-44) Fire Hose Station M-11 should be changed to N-11. This is a typographical change.
- 4) In surveillance 4.8.1.1.2f 6b (pg 3/4 8-5) an "s" must be added to the word "operate" to make the sentence grammatically correct.
- 5) In LCO 3.11.1.4 (pg 3/4 11-8) a comma should be added after "gases" to clarify the sentence.
- 6) In Section 6.5.2b 9 (pg 6-13) "shall submit" should be changed to "submittal of" to improve the grammar of the sentence.

TABLE 3.3-3

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

FUNCTIONAL UNIT	TOTAL NO. OF CHANNELS	CHANNELS TO TRIP	MINIMUM CHANNELS OPERABLE	APPLICABLE MODES	ACTION
1. Safety Injection (Reactor Trip, Feedwater Isolation, Start Diesel Generators, Containment Cooling Fans, Control Room Isolation, Phase "A" Isolation, Turbine Trip, Auxiliary Feedwater, Containment Vent Isolation, and Essential Service Water).					
a. Manual Initiation	2	1	2	1, 2, 3, 4	18
b. Automatic Actuation Logic and Actuation Relays	2	1	2	1, 2, 3, 4	14
c. Containment Pressure-High-1	3	2	2	1, 2, 3	15*
d. Pressurizer Pressure-Low (Above P-7) 	4	2	3	1, 2, 3#	19*
e. Steam Line Pressure-Low (Above P-11)	3/steam line	2/steam line any steam line	2/steam line	1, 2, 3#	15*
2. Containment Spray					
a. Manual Initiation	2 pair	1 pair	2 pair	1, 2, 3, 4	18
b. Automatic Actuation Logic and Actuation Relays	2	1	2	1, 2, 3, 4	14
c. Containment Pressure-	4	2	3	1, 2, 3	16

CONTAINMENT SYSTEMS

AIR TEMPERATURE

LIMITING CONDITION FOR OPERATION

3.6.1.5 Primary containment average air temperature shall not exceed 120°F.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With the containment average air temperature greater than 120°F, reduce the average air temperature to within the limit within 8 hours, or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.5 The primary containment average air temperature shall be the arithmetical average of the temperatures of the running fans at the following locations and shall be determined at least once per 24 hours:

Location

- A/ RCFC Dry Bulb Inlet Temperature
- B/ RCFC Dry Bulb Inlet Temperature
- C/ RCFC Dry Bulb Inlet Temperature
- D/ RCFC Dry Bulb Inlet Temperature.

TABLE 3.7-5b (Continued)
(Unit 2)
FIRE HOSE STATIONS

LOCATION	ELEVATION	HOSE RACK REEL	ANGLE VALVE
<u>Aux. Bldg. (Continued)</u>			
V-19: By West Door to Decon Pad & Storage	430	59	OFP344
Q-26: Back of Div. 21 SWGR Room	430	284	OFP641
S-21: By U-2 Pzr. HTR. Transformer (elec. pen. area)	419	175	OFP347
Q-26: By U-2 Elect. Penetration Area	419	206	OFP346
L-11: By waste oil tank room	405	90	OFP315
P-18: By elevator	405	91	OFP318
P-23: By spent resin pumps	405	92	OFP349
Q-11: By laundry tanks	405	93	OFP314
S-21: East of U-2 hydrogen recombiner	405	94	OFP348
V-21: West of U-2 hydrogen recombiner	405	95	OFP345
V-15: West of U-1 hydrogen recombiner control panel	405	96	OFP316
S-15: East of U-1 hydrogen recombiner	405	97	OFP317
N-11: By the recycle holdup tanks	368	130	OFP373
M-13: By the U-1 stirrs	368	131	OFP374
P-13: By panel 1PL84JB	368	132	OFP369
L-20: By the U-2 stairs	368	133	OFP355
P-21: By the blowdown condenser	368	134	OFP356
L-25: By the PW M/U pumps	368	135	OFP361
N-25: By chemical drain tank	368	136	OFP357
S-18: By panel 1PL86J	368	138	OFP362
Q-11: By Aux. Bldg. floor drain tanks	368	139	OFP368
Q-25: By spent resin flushing pump	368	137	OFP360
U-21: By U-2 spray add tank	368	142	OFP358
V-18: By U-2 cent. chg. pump room	368	141	OFP366
P-11: By recycle evaporator feed pumps	350	151	OFP381
M-13: By U-1 stairs	350	152	OFP370
N-23: By gas decay tanks	350	154	OFP352
Q-19: By "B" Aux. Bldg. Equip. drain tank	350	155	OFP365
Q-17: By "A" Aux. Bldg. Equip. drain tank	350	156	OFP371

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- 3) Verifying the diesel generator capability to reject a load of 5500 kW without tripping. The generator voltage shall not exceed 4784 volts during and following the load rejection,
- 4) Simulating a loss of ESF bus voltage by itself, and:
 - a) Verifying de-energization of the ESF busses and load shedding from the ESF busses, and
 - b) Verifying the diesel starts on the auto-start signal, energizes the ESF busses with permanently connected loads within 10 seconds, energizes the auto-connected safe shutdown loads through the load sequencing timer and operates for greater than or equal to 5 minutes while its generator is loaded with the shutdown loads. After energization, the steady-state voltage and frequency of the ESF busses shall be maintained at 4160 ± 420 volts and 60 ± 1.2 Hz during this test.
- 5) Verifying that on an ESF Actuation test signal without loss of ESF bus voltages, the diesel generator starts on the auto-start signal and operates on standby for greater than or equal to 5 minutes. The generator voltage and frequency shall be 4160 ± 420 volts and 60 ± 1.2 Hz within 10 seconds after the auto-start signal; the generator steady state generator voltage and frequency shall be maintained within these limits during this test;
- 6) Simulating a loss of ESF bus voltage in conjunction with an ESF Actuation test signal, and
 - a) Verifying deenergization of the ESF busses and load shedding from the ESF busses;
 - b) Verifying the diesel starts on the auto-start signal, energizes the ESF busses with permanently connected loads within 10 seconds, energizes the auto-connected emergency (accident) loads through the LOCA sequencer and operate for greater than or equal to 5 minutes while its generator is loaded with emergency loads. After energization, the steady-state voltage and frequency of the ESF busses shall be maintained at 4160 ± 420 volts and 60 ± 1.2 Hz during this test; and
 - c) Verifying that all automatic diesel generator trips, except engine overspeed and generator differential, are automatically bypassed upon loss-of-voltage on the emergency bus concurrent with a Safety Injection Actuation signal.

RADIOACTIVE EFFLUENTS

LIQUID HOLDUP TANKS

LIMITING CONDITION FOR OPERATION

3.11.1.4 The quantity of radioactive material, excluding tritium and dissolved or entrained noble gases, contained in any outside tanks shall be limited to the following:

- a. Primary Water Storage Tank \leq 2000 Curies, and
- b. Outside Temporary Tank \leq 10 Curies.

APPLICABILITY: At all times.

ACTION:

- a. With the quantity of radioactive material in any of the above listed tanks exceeding the above limit, immediately suspend all additions of radioactive material to the tank, within 48 hours reduce the tank contents to within the limit, and describe the events leading to this condition in the next Semiannual Radioactive Effluent Release Report, pursuant to Specification 6.9.1.7.
- b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.11.1.4 The quantity of radioactive material contained in each of the above tanks shall be determined to be within the above limit by analyzing a representative sample of the tank's contents at least once per 7 days when radioactive materials are being added to the tank.

ADMINISTRATIVE CONTROLS

ONSITE (Continued)

- 3) Review of all proposed changes to the Technical Specifications;
- 4) Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety;
- 5) Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Division Vice President and General Manager - Nuclear Stations and to the Supervisor of the Offsite Review and Investigative Function;
- 6) Review of all REPORTABLE EVENTS;
- 7) Performance of special reviews and investigations and reports thereon as requested by the Supervisor of the Offsite Review and Investigative Function;
- 8) Review of the Station Security Plan and implementing procedures and submittal of recommended changes to the Division Vice President and General Manager - Nuclear Stations;
- 9) Review of the Emergency Plan and station implementing procedures and ~~shall submit~~ recommended changes to the Division Vice President and General Manager - Nuclear Stations;
- 10) Review of Unit operations to detect potential hazards to nuclear safety;
- 11) Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Division Vice President and General Manager - Nuclear Stations and the Supervisor of the Offsite Review and Investigative Function; and
- 12) Review of changes to the PROCESS CONTROL PROGRAM, the OFFSITE DOSE CALCULATION MANUAL, and the Radwaste Treatment Systems.

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c. Authority

The Technical Staff Supervisor is responsible to the Station Superintendent and shall make recommendations in a timely manner in all areas of review, investigation, and quality control phases of plant maintenance, operation, and administrative procedures relating to facility operations and shall have the authority to request the action necessary to ensure compliance with rules, regulations, and procedures when in his opinion such action is necessary. The Station Superintendent shall follow such recommendations or select a course

ATTACHMENT C

SIGNIFICANT HAZARDS CONSIDERATION

Commonwealth Edison has evaluated the proposed amendment and has determined that it does not represent a significant hazards consideration. Based on the criteria for defining a significant hazards consideration established in 10 CFR 50.92, operation of Byron Station Unit 1 in accordance with the proposed amendment will not:

- 1) Involve a significant increase in the probability or consequences of an accident previously evaluated because the proposed changes are purely administrative and only involve corrections of typographical errors and improvements in grammar and punctuation.
- 2) Create the possibility of a new or different kind of accident from any accident previously evaluated because the proposed changes are purely administrative and only involve corrections of typographical errors and improvements in grammar and punctuation.
- 3) Involve a significant reduction in a margin of safety because the proposed changes are purely administrative and only involve corrections of typographical errors and improvements in grammar and punctuation.

Based on the preceding assessment, it is concluded that the proposed amendment meets the standards provided in 10 CFR 50.92 and therefore, does not constitute a significant hazards consideration.