PDR

November 13, 1985

For: The Commissioners

From: T. A. Rehm, Assistant for Operations, Office of the EDO

Subject: WEEKLY INFORMATION REPORT - WEEK ENDING NOVEMBER 8, 1985

A summary of key events is included as a convenience to those Commissioners who may prefer a condensed version of this report.

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*No input this week.

T. A. Rehm, Assistant for Operations
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HIGHLIGHTS OF WEEKLY INFORMATION REPORT

WEEK ENDING NOVEMBER 8, 1985

Grand Gulf, Unit 1

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Grand Gulf Unit 1 is in a planned outage which began October 12 and is scheduled to end November 24. One of the tasks being performed is Induction Heating Stress Improvement (IHSI) of selected welds in the recirculation piping, including the welds adjacent to these crack indications. Licensee believes the crack indications are associated with, and may have been caused by, the IHSI process. Licensee is evaluating the situation, including grinding of the crack indications.

Microbiologically-induced Corrosion - Comanche Peak 1 and 2

Evidence of microbiologically-induced corrosion has been found in the fire protection system, particularly at fittings. Some pipe/fitting replacement may be required. The system is currently filled with water from the Squaw Creek Reservoir, which is high in dissolved minerals.

Texas Utilities Generating Company is investigating the extent to which the problem may exist in other systems. Colonies of bacteria have been found in the reactor water makeup tank for Unit 2. Unit 2 demineralized water sources are separate from Unit 1; no problems on Unit 1 have been found thus far. The problem is believed to be confined to stagnant systems. The Applicant will be issuing a report in the near future.

Dry Spent Fuel Storage

As reported previously, during an unlicensed demonstration at the Idaho National Engineering Laboratory (INEL), cracks were observed in non-structural welds in the fuel basket of the General Nuclear Systems, Inc. (GNSI) CASTOR V/21 nodular cast iron dry spent fuel storage cask. On October 31, 1985, a meeting was held between GNSI and NMSS staff. GNSI examination and analyses to date suggest that the cracks were caused by thermal stress due to storing fuel with a heat load (about 28.4 kW) beyond the design basis load (21 kW) in a basket fabricated with less margin for expansion (i.e., a tighter fit to attain greater thermal efficiency and to permit precise positioning of thermocouple leads for the test) than specified in the CASTOR V/21 design submitted to NRC. GNSI will submit a preliminary report concerning the INEL occurrence in November 1985. NRC staff will examine it and meet again with GNSI. Assuming resolution of this matter, GNSI will submit a second and probably final report to NRC. At this time it appears the cracks do not compromise the basket storage capability or safety and that future basket fabrication tolerances can be assured to eliminate the problem.

OFFICE OF ADMINISTRATION

Week Ending November 8, 1985

ADMINISTRATION OF THE FREEDOM OF INFORMATION ACT

STATUS OF REQUESTS

	Initial Request	Appeal of Initial Decision
Carryovers, 1984	179	23
Received, 1985	740	41
Granted	584	30
Denied	170	16
Pending	165	18

ACTIONS THIS WEEK

Received

Susan L. Hiatt, OCRE (85-728)	Requests copies of all records regarding the meeting between NRC and the Hydrogen Control Owners Group held on or about September 5, 1985.
Ellyn R. Weiss, Harmon, Weiss & Jordan (85-729)	Requests copies of the NRC's inspection report and related records regarding the system evaluation of the auxiliary feedwater system at Turkey Point Units 3 and 4.
Ron Kucera, State of Missouri (85-730)	Requests a copy of the license application filed by R.M. Wester and Associates and subsequent correspondence between the applicant and NRC.
John Harrison, The Columbian (85-731)	Requests DOE/NRC Forms 741 regarding shipments of nuclear fuel entering the US via Oregon and Washington from 1970 through October 1985.
(An individual requesting records) (85-732)	Requests records relating to an incident at Kelly Air Force Base on July 19, 1985.
Cindee Virostek, Kiski Valley Coalition (85-733)	Requests copies of specified records relating to B&W facilities in Apollo and Parks Township, Pennsylvania.
Joel M. Kaplan, Karlin and Fleisher (85-734)	Requests copies of procedures, rules, and regulations in effect in 1973 through 1976 regarding radiation exposure of persons working in the Zion nuclear power plant.

CONTACT: J. M. Felton
492-7211

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Received, Cont'd

John E. Ryan (85-735)	Requests a copy of OI's report on Nine Mile Point Unit 2 relating to nuclear auditor harassment.
Susan L. Hiatt, OCRE (85-736)	Requests copies of four categories of records identified in Enclosure E of NRC's 9/27/85 Weekly Information Report.
Steven Aftergood, Committee to Bridge the Gap (85-737)	Requests all records relating to an IAEA meeting on October 9-11, 1985, in Vienna, Austria, "IAEA Technical Committee Meeting on Preparation of Guidebooks on Conversion of Research Reactors from High to Low Enriched Uranium Fuels."
Steven Aftergood, Committee to Bridge the Gap (85-738)	Requests any written materials submitted to the NRC in response to the show cause orders by any of the 31 affected licensees or other interested persons regarding why excess HEU fuel should not be removed from research and test reactors and critical experimental facilities.
Donald K. Slingsby, Proto-Power/Bisco Nuclear, Inc. (85-739)	Requests copies of four specified records relating to Alaron, License No. 32-20826-01.
James F. Nicolosi, Hydro Nuclear Services (85-740)	Requests copies of all records relating to inspections in October 1984, January 1985, and June 1985, and any subsequent inspections regarding North American Inspection Inc.

Granted

Stephen H. Hanauer, Technical Analysis Corporation (85-481)	In response to a request for all training manuals and other technical references and instructional records used at the NRC Training Center, made available 30 records. Informed the requester that some training manuals were already available at the PDR.
Richard E. Webb, Studies of Nuclear Hazards and Constitutional Law (85-611)	In response to a request for 14 categories of records relating to the Davis-Besse nuclear power plant, informed the requester that 18 records subject to this request are already available at the PDR.
Charles Barnes (85-644)	In response to a request for all records from June 25, 1975, to March 1, 1976, relating to a waste shipment at the Beatty, Nevada, burial site from the Cimarron plutonium facility, as described in IE Bulletin 75-07, made available five records.

ENCLOSURE A

Granted, Cont'd

Frank Ruswick, West Michigan Environmental Action Council (85-650)	In response to a request for three categories of records relating to a request by Consumers Power Company to leave in place radioactively contaminated soil at the Big Rock Point nuclear power plant, made available four records. Informed the requester that five additional records subject to this request are already available at the PDR.
Patricia Hainer (85-681)	In response to a request for records concerning Serono Diagnostics, License No. 201593001, made available 14 records.
Louise Ponce (85-684)	In response to a request for eight categories of records relating to radioactive materials and waste at the Stepan Chemical Company in Maywood, New Jersey, made available six records. Informed the requester that four additional records subject to this request are already available at the PDR.
Lila Pope (85-702)	In response to a request for copies of records concerning each misadministered dose from New Center Radiology, Detroit, Michigan, informed the requester that the NRC could not locate any records subject to this request.
David S. Palmer, ERP&M (85-707)	In response to a request for copies of "Evacuation Time Estimates for the Plume Exposure Pathway EPZ at TMI," dated March 3, 1983, any amendments or updates, and all appendices or addendum to the report or its amendments or updates, made available a copy of the requested report. Informed the requester that NRC could not locate any other records subject to this request.

Denied

Scott Faust, The Wichita Eagle-Beacon (85-701)	In response to a request for a copy of the transcript of the Commission briefing held on June 3, 1985, which preceded the licensing hearing for the Wolf Creek nuclear power plant, informed the requester that two transcripts are already available at the PDR. Denied portions of one transcript pursuant to Exemptions 5 and 7 of the Government in the Sunshine Act.

WEEKLY INFORMATION REPORT
DIVISION OF CONTRACTS
WEEK ENDING NOVEMBER 8, 1985

RFP ISSUED

RFP No.: RGI-86-311
Title: "Technical Support for Non-Destructive Examination Van"
Description: On a task order basis, the contractor shall provide personnel with expertise in the field of non-destructive examination and a radiographic isotope source to perform field non-destructive examinations at nuclear power plants under construction or in operation anywhere in the United States.
Period of Performance: Twenty-four months
Sponsor: Region I, Division of Reactor Safety
Status: RFP issued on November 1, 1985. Proposals due on December 2, 1985.

RFP No.: RS-ORM-85-335
Title: "Integrated Library System"
Description: Software maintenance, support and timesharing services for the Integrated Library System (ILS) as installed and subsequently enhanced.
Period of Performance: 2 years with 1 option year
Sponsor: Office of Resource Management
Status: RFP issued on November 6, 1985. Proposals due December 6, 1985.

RFP No.: RS-ORM-85-336
Title: "ADP Information Technology Support Center Contract"
Description: End user computing support services for users of NRC microcomputers and NRC users of the NIH computer facility.
Period of Performance: 2 years
Sponsor: Office of Resource Management
Status: RFP issued on November 1, 1985. Proposals due December 3, 1985.

PROPOSALS UNDER EVALUATION

RFP No.: RS-RES-86-104
Title: "Cooperative Human Reliability Evaluation Program"
Description: Compare and contrast NRC and EPRI analytic tools for conducting human reliability analysis with those of other participating nations to assess the current state-of-the-art and to guide future research.
Period of Performance: 12 months
Sponsor: Office of Nuclear Regulatory Research
Status: RFP closed on October 30, 1985. Proposals forwarded to Source Evaluators for review on October 31, 1985.

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PROPOSALS UNDER EVALUATION (cont'd)

RFP No.: RS-ADM-85-235
Title: "NRC Translation Services"
Description: The contractor will be required to furnish translation services for reports and other related material provided by NRC when issued by a formal work order.
Period of Performance: 2 years
Sponsor: Office of Administration/Technical Information & Document Control
Status: Negotiations completed on November 5, 1985. Best and final offers due on November 12, 1985.

RFP No.: RS-OIE-85-159
Title: "NRC Training Courses for Inspectors and Engineers"
Description: The contractor will provide experienced instructors and other experts to conduct two training courses of 80 hours duration each. One course shall be titled, "Electrical Technology and Codes," the other "Instrumentation Technology Codes."
Period of Performance: One year with 3 one year options
Sponsor: Office of Inspection & Enforcement
Status: Best and Final Offers received on November 7, 1985 and forwarded to Source Evaluators for review on November 7, 1985.

CONTRACT AWARDED

RFP No.: RS-ADM-86-211
Title: "Newspaper Clipping Service"
Description: The contractor shall provide relevant news articles from local and national newspapers for inclusion in the NRC "Media Monitor".
Period of Performance: 2 years
Sponsor: Office of Administration
Status: Fixed Price Requirements Contract No. NRC-10-86-211 awarded to Press Intelligence, Inc. in the amount of $69,120.00, effective November 1, 1985.

CANCELLATION

RFP No.: RS-NMS-85-001
Title: "Development of Regulatory Effectiveness Review (RER) Team Training Guides"
Description: Development of detailed guidance and instructional materials to be used in training NRC staff to conduct safeguards regulatory effectiveness reviews.
Period of Performance: 2 years
Sponsor: Office of Nuclear Material Safety and Safeguards
Status: RFP cancelled on November 7, 1985.

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ADMINISTRATIVE MATTER

The protest filed by Compucom Security, Inc., on August 16, 1985 against revised RFQ 0001, Mod. 1 was dismissed with prejudice by the General Services Board of Contract Appeals on October 22, 1985. NRC and Compucom settled the protest and jointly requested dismissal of the protest.

OFFICE OF NUCLEAR REACTOR REGULATION

ITEMS OF INTEREST

Week Ending November 8, 1985

Fort Calhoun Station Unit No. 1

The Fort Calhoun Station is currently in a refueling outage. The licensee just completed eddy current testing of the steam generators tubes. The licensee tested 958 tubes in the A steam generator and 965 tubes in the B steam generator. Eighteen tubes in the A steam generator and seventeen tubes in the steam generator were considered to be degraded; no tubes were considered to be defective. All but three tubes were considered degraded because the eddy current probe would not pass thru the tubes.

The licensee eddy current tested 9925 tubes out of 10,010 tubes in mid-1984. Only nine tubes were considered degraded because the probe would not pass.

The significance of the above information is that denting appears to be progressing in the Fort Calhoun Station steam generators. Although continued denting could result in leaking tubes, it is not expected to result in a tube rupture.

Grand Gulf, Unit 1

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ENCLOSURE B

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Limerick

On October 28, 1985 the Licensing Board issued an Order in response to the Appeal Board's remand of the PID-2 onsite emergency planning back-up medical arrangements issue. The Order requires an applicant response to the issue by November 18, a response by the intervenor and the Commonwealth of Pennsylvania by December 2 and a response by the staff by December 12. The Order did not specify any other action other than submittal of the responses.

On October 24, 1985 the Appeal Board issued an Order that specified that oral argument on the appeals by intervenors Air and Water Pollution Patrol and the Graterford inmates from the Fourth Partial Initial Decision will be held on December 4, 1985. PID-4 concerned off site emergency planning for the inmates at the State Correctional Institute at Graterford, PA.

Limerick Unit 1 is at 69% (664 MWe) power and is conducting feedwater system tuneups.

Rancho Seco Startup

The Rancho Seco Nuclear Generating Station attained criticality at 10:48 p.m. PST on November 2, 1985, and as of 11:50 EST on November 4, 1985, the plant was stabilized at 43% power. The plant will remain at approximately 40% power for several days while auxiliary boiler repairs are in progress. A main turbine generator trip test will be performed prior to increasing power to 100%.

The startup followed resolution by the Sacramento Municipal Utility District (SMUD) of safety concerns raised by NRR and Region V. The NRC safety concerns were raised following the analyses of plant transients which occurred during an unplanned shutdown from 15% power on October 2, 1985. Prior to restart, SMUD was required to modify the automatic start circuitry for the auxiliary feedwater (AFW) system. The AFW initiation circuitry modification was required after it was determined that during the October 2, 1985 transients, manual action by the operators was necessary to initiate AFW flow.

In addition to AFW system hardware modifications, SMUD completed evaluations to resolve other shutdown related concerns. These involved the main feedwater system, secondary plant vacuum, the high pressure injection system, and primary plant cooldown. SMUD also committed to expedite AFW related Technical Specification changes and to evaluate the feasibility of accelerating the schedule for upgrading the AFW control system.

Comanche Peak 1 and 2

Microbiologically-induced Corrosion

Evidence of microbiologically-induced corrosion has been found in the fire protection system, particularly at fittings. Some pipe/fitting replacement may be required. The system is currently filled with water from the Squaw Creek Reservoir, which is high in dissolved minerals.

Texas Utilities Generating Company is investigating the extent to which the problem may exist in other systems. Colonies of bacteria have been found in the reactor water makeup tank for Unit 2. Unit 2 demineralized water sources are separate from Unit 1; no problems on Unit 1 have been found thus far. The problem is believed to be confined to stagnant systems. The Applicant will be issuing a report in the near future.

Asiatic Clams

Asiatic clams are now present in the Squaw Creek Reservoir. They have been found in the service water intake structure and in one component cooling heat exchanger (which has not been in service recently). The service water system has been realigned to eliminate any stagnant portions. The Applicant is updating its response to the I&E Bulletin on this subject.

Point Beach Nuclear Power Plant, Unit No. 2

The staff was notified that during visual inspection of fuel assemblies removed from the core during the current refueling outage, the licensee identified two fuel assemblies in a degraded condition. Some fuel rods showed signs of erosion and at least one fuel rod was severed. In the case of the severed fuel rod, fuel pellets were released.

The licensee determined that the fuel rod problems were due to baffle jetting similar to that experienced previously on Unit 1. The licensee is evaluating corrective actions but the corrective actions will not be completed until the next outage. The staff will evaluate whether it is acceptable to resume operations before all the corrective actions are completed.

St. Lucie Plant, Unit No. 1

At approximately 8:30 a.m. on November 6, 1985, we were informed that activity associated with the St. Lucie Plant, Unit No. 1 Cycle 7 refueling outage had been stopped. During the night the lifting device was attached to the upper guide structure and the lift was started. Before completing the lift it was discovered that the upper guide structure was tilted. Investigation revealed that one of the three bolts used to secure the upper guide structure to the lifting device was not properly secured and the upper guide structure was being supported by only two bolts. Based upon our concern that failure of these two bolts might allow the upper guide structure to fall causing fuel damage, the licensee issued an unusual event report at 10:45 a.m., pending the results of an engineering analysis of the situation.

In a 3:00 p.m. conference call that involved the site management and Region II, the licensee informed the NRC that he is in process of developing an independent system to support and remove the upper guide structure and attached lifting device. The design of this independent support system is expected from the NSSS vendor within 1 to 2 hours of the conference call. The two engaged bolts appear to be fully engaged and the upper guide structure is free to move within the vessel. Engagement of the bolts is based upon torque and a faulty torque reading could have been indicated due to cross-threading, etc. The independent system is supposed to be designed by the NSSS vendor to lift the upper guide structure and attached lifting device from its current location and remove it from the vessel.

This occurrence will cause a further delay in the outage schedule that cannot be estimated until the design of the new support system is known and fabricated. At a minimum it is expected to be 1 1/2 to 2 days. An earlier delay of 4 days was caused by problems associated with installation of nozzle dams.

Sequoyah Nuclear Plant, Units 1 and 2

On November 4, 1985, TVA issues a summary of their November 4, 1985 response to the 10 CFR 50.54(f) letter regarding corporate and Sequoyah-related concerns. TVA stated the TVA Board's philosophy and indicated the following actions will be taken to ensure the TVA Board maintains a close awareness of nuclear activities:

a. the acquisition of a senior nuclear advisor to advise the Board specifically on nuclear matters

b. the use of frequent special Board briefings dedicated solely to the nuclear program

c. the assignment of NSRS senior representatives full time to each plant site

d. the increase of support for the nuclear program by other TVA organizations (General Counsel, Personne, etc.)

e. the creation of the Office of Inspector General by December 1, 1985

f. the improvement in compensation to attract and retain experienced personnel

In addition, TVA stated that is is establishing a Senior Management Team (headed by H. Parris, Manager of Power and Engineering (Nuclear) to establish the duties, responsibilities, and authority for each organization in the TVA nuclear program, and listed other corporate goals.

The licensee is committing to use project managers and program managers to ensure accountability on resolution of key issues and implementation of major efforts. In addition, TVA is setting up a Corporate Commitment Tracking System to replace the (at least) seven separate commitment tracking system utilized by different TVA organizations.

With regard to Sequoyah-specific concerns TVA stated their submittal fully describes the EQ program at the facility, with a commitment to verify component location and provide documentation that the equipment is qualified prior to restart.

In response to a concern regarding cable tray support analyses, TVA has performed an evaluation to show that this design program meets all applicable design requirements. In addition, TVA is performing a design control survey for Sequoyah scheduled to be completed in November.

Finally, the licensee stated they are providing information regarding the operational readiness of Sequoyah.

Peach Bottom Atomic Power Station, Unit 3

On September 20, 1985, a Daily Highlight was issued pertaining to the "identification of cracks" by Philadelphia Electric Company (PECo or the licensee) at Peach Bottom, Unit 3, on the 28-inch recirculation outlet safe end to pipe welds. It was indicated at that time that the safe ends were composed of 316L, low carbon material.

On October 31, 1985, the licensee met with the NRC staff to discuss its findings on these cracks based upon core SAMPLE metallography. The licensee in this presentation concluded that there is no metallographic evidence of IGSCC cracking in the low carbon 316L safe ends and that available evidence indicates that the reported UT indications of cracks are believed to be in result of unique weld root geometry and the presence of small weld defects.

This evidence presented by the licensee is currently under staff review. PECo has indicated that the anormalies presented by this unique weld geometry will be the subject of discussions during a BWR Owner's group meeting scheduled for November 14, 1985.

NRC TMI PROGRAM OFFICE WEEKLY STATUS REPORT

WEEK ENDING NOVEMBER 8, 1985

1. DEFUELING

Six entries were made during the week for movement of fuel debris. The objective was to excavate an area in the debris bed such that the Canister Positioning System (CPS), when loaded with defueling canisters, can rotate without interference. Approximately 2,000 lbs. of debris, consisting mainly of end fittings and attached fuel rods, were moved from the northern quadrants within the reactor vessel to the southern regions. By the end of the final defueling entry this week, it appeared that the objective had been accomplished. During next week's entry for defueling, video inspections will be made to verify the clearance for the CPS. Any remaining partial assemblies or rods that may still be sticking up above the debris bed (308' elevation) will be removed.

The average dose rate to each defueling worker was about 25 to 35 mrem per entry or about 10 mrem per hour. This indicates that fuel movement has not increased the dose rates in areas of the building occupied by defueling workers. Air samples also indicate no significant increase in airborne activity. The NRC is closely monitoring the defueling activities to ensure that operations are conducted safely in accordance with applicable procedures.

2. PLANT STATUS

- The facility remains in long term cold shutdown with the Reactor Coolant System (RCS) vented to the reactor building atmosphere and the reactor vessel head and plenum assembly removed from the reactor vessel.
- The plenum is on its storage stand in the deep end of the fuel transfer canal. A dam has been installed between the deep and shallow ends of the fuel transfer canal. The deep end is filled with water to a depth of about 20 feet (about 5 feet above the top of the plenum).

- The modified internals indexing fixture is installed on the reactor vessel flange and is flooded to elevation 327 feet 6 inches (15½ feet above the top of the core region). The defueling platform is installed over the Internal Indexing Fixture in preparation for defueling.
- Calculated reactor decay heat is less than 12 kilowatts.
- RCS cooling is by natural heat loss to the reactor building ambient atmosphere. Incore thermocouple readings range from 71°F to 95°F with an average of 83°F.
- The average reactor building temperature is 58°F. The reactor building airborne activity at the Westinghouse platform is 1.8 E-7 uCi/cc Tritium and 1.1 E-9 uCi/cc particulate, predominantly Cesium 137.
- Spent Fuel Pool "A" is flooded to a depth of 20 feet. About 6 feet of water is over fuel canister storage racks.

3. WASTE MANAGEMENT

- The Submerged Demineralizer System (SDS) completed processing batch 125, Fuel Transfer Canal through Train No. 1. A total of 203,369 gallons was processed in Batch 125. Processing of batch 126 commenced, Fuel Transfer Canal recycle through both Trains and "B" cation sand filter.
- EPICOR II is temporarily shutdown while changing out liners.
- Total volume processed through SDS to date is 3,174,794 gallons, and the total volume processed through EPICOR II is 2,700,737 gallons.

4. DOSE REDUCTION/DECONTAMINATION ACTIVITIES

- Decontamination activities are continuing on the 281' level of the auxiliary building. Scabbling of reactor coolant bleed tank cubicles is in progress.
- Average general area radiation dose rate is 40 mrem per hour on the 347' level of the reactor building and is 67 mrem per hour on the 305' level of the reactor building.
- Decontamination of the pressurizer and "A" D-ring is in progress.

5. ENVIRONMENTAL MONITORING

- US Environmental Protection Agency (EPA) sample analysis results show TMI site liquid effluents to be in accordance with regulatory limits, NRC requirements, and the City of Lancaster Agreement.
- TMI water samples taken by EPA at the plant discharge to the river consisted of seven daily composite samples taken from October 20 through October 26, 1985. A gamma scan detected no reactor related activity.
- The Lancaster water sample taken at the water works intake and analyzed by EPA consisted of a seven day composited sample taken from October 20 through October 26, 1985. A gamma scan detected no reactor related radioactivity.

- The NRC outdoor airborne particulate sampler at the TMI Site collected a sample between October 30 and November 7, 1985. No reactor related radioactivity was detected. Analysis showed Iodine-131 and Cesium-137 concentrations to be less than the lower limits of detectability.

6. REACTOR BUILDING ACTIVITIES

- The initial phase of defueling the reactor core is in progress.
- Defueling Water Cleanup System (DWCS) preoperational testing and modification continued.
- Installation of the vacuum defueling system is in progress.
- Work is in progress on the canister positioning system.
- An integrated test of the canister handling bridges of the Reactor Building and the Fuel Handling Building and the Fuel Transfer Canal is in progress. This will be accomplished by transfer of a canister from the Fuel Transfer Canal to the Spent Fuel Pool through each of the Fuel Transfer Tubes.

7. AUXILIARY AND FUEL HANDLING BUILDING ACTIVITIES

- Installation of the DWCS continued. Partial DWCS turnover for processing RCS during early defueling is expected to be completed in October.
- Spent Fuel Pool has been flooded to a depth of about 20 feet (about 6 feet above the top of the fuel canister storage racks).

8. NRC EVALUATIONS IN PROGRESS

- Technical Specification Change Request number 49.
- Recovery Operations Plan Change number 31.
- Defueling Safety Evaluation.
- SDS Technical Evaluation and System Description Update.
- Core Stratification Sample Safety Evaluation.
- Heavy Load Handling Safety Evaluation Report.
- Defueling Water Cleanup System Technical Evaluation Report, Revision 7.
- Containment Air Control Envelope Technical Evaluation Report, Revision 5.

9. PUBLIC MEETING

The next meeting of the Advisory Panel is scheduled for 11:00 AM, November 19, 1985, in Washington, DC, before the NRC Commissioners. The next meeting in the TMI area is scheduled for December 12, 1985, at the Harrisburg, PA Holiday Inn, 23 South Second Street, Harrisburg, PA, from 7:00 PM to approximately 10:00 PM.

Persons desiring the opportunity to speak before the Panel are asked to contact Mr. Thomas Smithgall at 717-291-1042 or write to him at 2122 Marietta Avenue, Lancaster, Pennsylvania 17603.

ENCLOSURE B

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OFFICE OF NUCLEAR MATERIAL SAFETY AND SAFEGUARDS

Items of Interest

Week Ending November 8, 1985

Near Term NRC Actions Under the Nuclear Waste Policy Act (NWPA)

Section: 121(a) of NWPA: EPA Final High-Level Waste Standards

Status: EPA final rule for Environmental Standards for the Management and Disposal of Spent Nuclear Fuel, High-Level and Transuranic Radioactive Waste will become effective November 16, 1985.

Action: Staff will review NRC's high-level waste criteria for conformance with EPA standards and will provide a proposed rule to the Commission in December, 1985.

Section: 301 of NWPA. DOE Mission Plan

Status: NRC testified on the Mission Plan before the Senate Subcommittee on Nuclear Regulation on October 30, 1985.

NRC testified on November 6, 1985 before the House Subcommittee on Energy Research and Production concerning the implementation of the NWPA, the Mission Plan and the MRS.

Section: 306 of NWPA: Regulations for Training of Power Plant Personnel

Status: The review of the final rulemaking package on Part 55 dealing with simulator training requirements and three associated regulatory guides was completed at the end of October.

Action: The final rulemaking package will undergo Office review through November and is expected to be submitted to ACRS by late November for review.

Section: 141 of NWPA: Licensing of MRS

Status: NRC is currently developing revisions to 10 CFR Part 72 to provide the licensing framework for the MRS, should it be authorized by Congress. If authorized, NRC will review DOE's license application and make the necessary licensing determinations.

Action: CRGR is scheduled to be briefed concerning the proposed revisions on November 13, 1985.

ENCLOSURE C

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Section: 112(b) of NWPA: Site Nomination and Recommendation

Status: DOE announced on October 30, 1985, that the date for site nomination and recommendation with the accompanying EAs cited in the draft PDS as November 1985 has been extended by 60 days to accommodate for the NAS ranking methodology review. Nomination, recommendation, and final EAs are now expected by late February, 1986.

DOE's Proposed Rulemaking on the Definition of Byproduct Material

On November 1, 1985, the Department of Energy (DOE) proposed a rule intended to clarify the definition of the term "byproduct material" as defined under Section 11e(1) of the Atomic Energy Act. The proposed interpretive rule is an effort by DOE staff to resolve jurisdictional disputes with EPA over the regulation of wastes at DOE facilities which contain both radioactive and hazardous constituents (referred to as "mixed" wastes). The stated purpose of the proposed rule is to determine DOE's obligations under the Resource Conservation and Recovery Act of 1976 (RCRA), and to determine which of the "mixed" wastes produced or owned by DOE should be subject to regulation under that act. While the rule is ostensibly written to specifically resolve only the DOE/EPA mixed waste regulatory questions, it could also impact on NRC licensing activities since it sets out a new approach for determining which substances are to be considered to be byproduct material.

Comments on the proposed rule are due to DOE on December 2, 1985. NRC staff plan to send comments to DOE and will keep the Commission informed.

Dry Spent Fuel Storage

As reported previously, during an unlicensed demonstration at the Idaho National Engineering Laboratory (INEL), cracks were observed in non-structural welds in the fuel basket of the General Nuclear Systems, Inc. (GNSI) CASTOR V/21 nodular cast iron dry spent fuel storage cask. On October 31, 1985, a meeting was held between GNSI and NMSS staff. GNSI examination and analyses to date suggest that the cracks were caused by thermal stress due to storing fuel with a heat load (about 28.4 kW) beyond the design basis load (21 kW) in a basket fabricated with less margin for expansion (i.e., a tighter fit to attain greater thermal efficiency and to permit precise positioning of thermocouple leads for the test) than specified in the CASTOR V/21 design submitted to NRC. GNSI will submit a preliminary report concerning the INEL occurrence in November 1985. NRC staff will examine it and meet again with GNSI. Assuming resolution of this matter, GNSI will submit a second and probably final report to NRC. At this time it appears the cracks do not compromise the basket storage capability or safety and that future basket fabrication tolerances can be assured to eliminate the problem.

NFS-Erwin

Strike by OCAW members continues. There has been some negotiating between the Union and NFS within the last four weeks but no resolution has been reached. NFS is conducting limited operations of the HEU production, scrap recovery and R&D facilities. No problems with site operations have arisen to date.

OFFICE OF INSPECTION AND ENFORCEMENT
Items of Interest
Week Ending November 8, 1985

1. The following Significant Enforcement Actions were taken during the past week:

   a. EN 85-76, a Notice of Violation and Proposed Imposition of Civil Penalty in the amount of $500 was issued November 7, 1985 to Quality Assurance Testing. This action is based on several violations, including the use of license material by technically unqualified individuals.

   b. EN-85-77, a Notice of Violation and Proposed Imposition of Civil Penalty in the amount of $37,500 was issued November 8, 1985 to Commonwealth Edison Company. This action is based on a violation involving the failure to maintain control over the security badge system.

2. The following IE Preliminary Notifications were issued during the past week:

   a. PNO-I-85-83, Duquesne Light Company (Beaver Valley Units 1 & 2), Unmonitored Release from Unit 1 Boric Acid Hold Tank.

   b. PNO-I-85-84, Niagara Mohawk Power Corporation (Nine Mile Point Unit 1), Unscheduled Outage/Emergency Tech Spec Amendment Needed for Restart.

   c. PNO-I-85-85, Rochester Gas and Electric Corporation (Ginna), Automatic Actuation of Cable Tunnel Deluge System.

   d. PNO-I-85-85A, Rochester Gas and Electric Corporation (Ginna), Update on Unusual Event Due to Fire System Actuation.

   e. PNO-II-85-103, U.S. Department of Energy (Oak Ridge, TN), Stolen Radioactive Material Shipping Packages.

   f. PNO-II-85-104, Babcock and Wilcox Company (Lynchburg Research Center, Lynchburg, VA), Onsite Flooding.

   g. PNO-II-85-105, Florida Power and Light Company (St. Lucie 1), Core Internals Upper Guide Structure Lifting Rig Failure.

   h. PNO-II-85-106, Mississippi Power and Light Company (Grand Gulf Unit 1), Engineer Injured While Inspecting Cooling Tower.

   i. PNO-II-85-107, Duke Power Company (Oconee Unit 1), Unscheduled Shutdown Greater Than 48 Hours (Update).

   j. PNO-III-85-93B, American Electric Power (D.C. Cook), Failure of Reactor Trip Breaker (Second Update).

   k. PNO-V-85-75, Sacramento Municipal Utility District (Rancho Seco), Resumption of Power Operations.

3. The following IE Information Notices and IE Bulletins were issued during the past week:

   a. IE Information Notice 85-86, Lightning Strikes at Nuclear Power Generating Stations was issued November 5, 1985 to all nuclear power reactor facilities holding an operating license or a construction permit.

   b. Bulletin 85-02, Undervoltage Trip Attachments of Westinghouse DB-50 Type Reactor Trip Breakers, was issued to all Power Reactor Licensees on November 5, 1985. The Bulletin is addressed for action to licensees of operating Westinghouse Power Reactors that have not yet installed the automatic shunt trip feature on reactor trip breakers. The basis for the Bulletin is the failure of one reactor trip breaker at D. C. Cook on October 29, followed by testing on November 3 identifying degradation of the breaker that functioned correctly on October 29.

4. Other Items

   b. Resident Inspector Seminar

   Director, IE met with principal staff in Region III November 5, 1985 to address the Resident Inspector Seminar.

   b. ACRS Brief

   Representatives of the Division of Emergency Preparedness and Engineering Response and NRR briefed the ACRS regarding Recent Significant Operating Events at Nuclear Power Plants on November 7, 1985.

   c. Maintenance Training Overview Class

   A representative of Operating Reactor Programs Branch, Division of Inspection Programs, attended a training class, "Maintenance Training Overview," at the GE BWR Services Training Facility in San Jose, CA, November 4-8, 1985.

   d. NFS License Renewal

   A representative of Special Materials Programs Branch, Division of Inspection Programs, participated in a meeting November 6, 1985 in Atlanta with the Region II staff and NMSS to discuss the license renewal for the NFS facility.

e. Hatch 1 Drywell Inerting and Purge Line Crack

Representatives of Events Analysis Branch, Division of Emergency Preparedness and Engineering Response were in Region II on November 8, 1985 to participate in a meeting with Georgia Power/GE to discuss the Hatch 1 drywell inerting and purge line crack of December 1984.

f. Fort Calhoun Installation Inspection

Representatives of Operating Reactor Programs Branch, Division of Inspection, and consultants were at the Fort Calhoun site November 4, 1985 to begin the installation inspection part of the trial outage inspection program.

g. Comanche Peak Inspection

Representatives of Reactor Construction Programs Branch, Division of Inspection were at Comanche Peak November 4, 1985 to discuss and observe Comanche Peak Response Team inspection activities.

h. Incident Response Program

Chief, Incident Response, Division of Emergency Preparedness and Engineering Response, held a briefing on November 6, 1985 for Public Affairs Officers on the agency's incident response program.

i. Reentry-Recovery Issues Conference

Director, and representatives of Division of Emergency Preapredness and Engineering Response attended a Reentry-Recovery Issues Conference November 7-8, 1985, which was a preparatory meeting for the Relocation Tabletop Exercise to be held in December. The Region I Administrator and staff also attended.

j. Systematic Assessment of Licensee Performance (SALP) Manual Chapter Revision

A revision to the SALP Manual Chapter was forwarded to the regional offices on November 5, 1985 for implementation. One important change is formal inclusion of Training and Qualification Effectiveness as a separate SALP functional area.

k. Inspection Program Revision

The Division of Inspection Programs issued on November 5, 1985 a revision to the Operating Reactor Inspection Program which provided guidance for allocating inspection resources and for development of site-specific inspection plans. A major purpose of the revision is to establish an approach for focusing inspection attention on poor licensee performance areas.

l. Special Safety System Functional Inspection

A special safety system functional inspection is in progress at Pilgrim with onsite inspection activities scheduled to be completed on November 22, 1985. Particular attention is directed to the details of modifications and design control, quality of maintenance and surveillance, and adequacy of testing applicable to the selected safety system. The first inspection of this type was conducted at Turkey Point in August/September.

m. Design Assurance Program

A representative of Division of Quality Assurance, Vendor, and Technical Training Center participated this week in the public meeting on Comanche Peak to make a presentation on the status of the IE overview of the applicant's Design Assurance Program.

n. Vendor Inspections

The following inspections were conducted this week:

(1) Limotorque, Lynchburg, VA - to review vendor design control activities and related concerns associated with PORV, valve/actuators, and magnesium rotors used in Reliance motors.

(2) Illinois Fabricators, Bradley, ILL - allegation inspection.

(3) Arkansas Nuclear One, Russellville, AK - to assist NRR in their survey of maintenance and surveillance at Arkansas Nuclear One Unit 2.

o. Civil Penalties Paid

(1) On November 1, 1985, payment in the amount of $2,000 was received from Princeton University (Princeton, NJ) for enforcement action (EA 85-70) relating to health physics violations.

(2) On November 8, 1985, payment in the amount of $50,000 was received from Arizona Public Service Company (Palo Verde) for enforcement action (EA 85-87) relating to violations involving the inoperability of the containment atmosphere portion of the PASS.

OFFICE OF NUCLEAR REGULATORY RESEARCH

Items of Interest

Week Ending November 8, 1985

2D/3D

Full-scale testing of emergency core cooling (ECC) performance is conducted in the international 2D/3D facilities Upper Plenum Test Facility (UPTF) and the Slab Core Test Facility (SCTF). Data is provided on fluid-fluid mixing (pressurized thermal shock) PTS related, ECC bypass, upper-plenum deentrainment, and fallback and condensation effects. The 2D/3D Program participants met in Bethesda, Maryland on October 28-31, 1985 to discuss the status of the program in each country and to resolve any outstanding issues. Also present at the meeting were personnel from NRR and the pressurized water reactor vendors (Westinghouse Electric Corporation, Combustion Engineering Incorporated, and Babcock and Wilcox Company).

The Japan Atomic Energy Research Institute reported that the test results from its Slab Core Test Facility, which simulates a radial segment of a PWR core, show that the simulated core with 2000 electrically heated rods was effectively cooled by the steam and water mixtures and quenched within 5 minutes. The quench pattern was nearly uniform (within 10 percent deviation) in the radial direction although the power level was different by as much as 40 percent; the high power zone being 20 percent higher than the average and the low power zone 20 percent lower. There seem to be significant flows in the horizontal direction, effectively mitigating the power level differences in the radial direction. It appears that the high power zone gets more emergency core cooling (ECC) flows for the following two reasons: (1) generation of more steam, thus inducing a higher buoyancy force, and (2) accumulation of less liquid above the upper tie-plate, thus providing a path of less resistance for ECC flows from the lower plenum to the upper plenum.

The heat transfer in the high power zone improves so much that the quench front in the higher power zone actually moves faster than the low power zone, although the difference is not significant (less than 10 percent).

ENCLOSURE E

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Semiscale

Semiscale Test S-NH-1 was successfully completed this week. Test S-NH-1 simulates a 0.5 percent cold-leg break (2 inch diameter) in a 4-loop, 3411 MW(t) commercial reactor in which the high pressure injection emergency core cooling system is inoperative. This break scenario in a commercial reactor has the potential for core dryout and heatup while still at high primary coolant pressures, which precludes accumulator injection. Thus it is necessary for operators to take timely and appropriate actions to reduce the primary coolant system pressure such that the accumulators and low pressure injection system (LPIS) can operate, assuring long-term core cooling before core degradation occurs. In this test, recovery operations were initiated when peak cladding temperature reached 1000°F. At that time the operators latched open the steam generator atmospheric dump valves, enabling the primary coolant system to depressurize as a result of the improved heat transfer to the secondary coolant system through an operation normally referenced to as feed and steam cooldown. This process was continued until the LPIS set point of 198 psia was reached.

International Code Assessment and Applications Program (ICAP)

A meeting of the participants in ICAP was held on October 21, 1985. Representatives from Austria, Belgium, Finland. France, Federal Republic of Germany, Italy, Netherlands, Spain, Sweden, Switzerland, United Kingdom, and the Joint Research Center Ispra Establishment attended, as well as the NRC and its contractors.

The program is concerned with the assessment, application, and improvement of the codes TRAC-PWR, RELAP5, and TRAC-BWR. The assessment effort includes separate effects and integral experiments, and applications of the codes to power plant transients. The formation of an international cooperative program was undertaken to utilize the considerable experience being gained through use of the codes in different countries. The program was organized during 1984-1985 through bilateral agreements between the NRC and the nuclear safety authorities of the different countries. An information meeting was hosted by the NRC in April 1985. The current meeting was aimed at the establishment of a Program Group, the finalization of principles by which the program will be managed, and the discussion of work plans for 1986.

Countries have begun generating information using the codes and providing results to the NRC. Quarterly newsletters prepared by the Los Alamos National Laboratory on TRAC-PWR and by the Idaho National Engineering Laboratory on RELAP5 and TRAC-BWR provide regular summaries of status and progress. The first ICAP specialist meeting is planned for the May-June 1986 time frame.

Degraded Core Coolability Experiment DCC-3 in ACRR

The last of three experiments on Degraded Core Coolability, experiment DCC-3, was performed successfully in the ACRR test reactor at Sandia National Laboratory. In the DCC experiments the limiting conditions were measured for which coolability can be restored and maintained by reflooding a severely damaged reactor core, as in the TMI-2 accident. The primary purpose of these experiments was to validate for LWR-specific accident conditions the relatively advanced core-debris coolability models that have been developed in LMFBR safety research. The LWR-specific conditions of importance are the pressure range up to 17.MPa (2,500 psi) for the water coolant, deep debris beds, variable inlet flow down to zero, and LWR specific debris characteristics. Experiment DCC-3 was entirely funded by NRC's foreign program partners in the joint international Severe Fuel Damage and Source Term Research Program.

In the DCC experiments, a particulate bed with the desired particle-size distribution is constructed from crushed and sieved reactor fuel (UO_2) in a water-filled experiment capsule, and is then internally fission-heated in the ACRR experiment cavity in simulation of the internal fission-product-decay heating of water-cooled core debris in a reactor accident. An array of 34 thermocouples throughout the bed detects the local dry-out coolability limit of the bed for a specific set of conditions by the ramp increase in the temperature of one or more of the bed thermocouples above saturation that follows dry out. Experiments DCC-1 and DCC-2, respectively, used a relatively fine and a relatively coarse particle size distribution in the bed and covered the full range of pressure. Experiment DCC-3 used a bed vertically stratified by particle size with the fine debris at the top, a configuration that occurs naturally from debris settling through water. Analysis and previous experiments have shown that this geometry can reduce the dry out coolability limit by as much as an order of magnitude under conditions of no inlet flow because of capillary pressure (the liquid is sucked towards the fine debris, reducing internal water flow to the bottom of the bed). DCC-3 also had capability for measuring the effect of inlet flow on the dry-out coolability limit.

The DCC-3 experiment was performed successfully with a 10-day run in ACRR. Fifty bed dryouts were measured that covered the planned flow and no-flow parameter range up to 7.0 MPa (1,000 psi). Beyond dryout behavior (bed thermal characteristics) was also measured up to 1300K. Preliminary results appear to be in agreement with the accepted theoretical model (Lipinski one-dimensional model), with the strong vertical stratification of the bed reducing the bed specific power required for dry out by about an order of magnitude from that for an unstratified bed. This large reduction, however, was eliminated by relatively small inlet flow from the bottom of the bed, as predicted, and the effect of inlet flow in general in increasing the specific power required to dry-out the bed was in agreement with the theoretical model.

Remotec Survey Robot (SURBOT) Scheduled For Demonstration at NRC

In 1984, the Office of Nuclear Regulatory Research, under the Small Business Innovative Research Act, funded a robotics demonstration project with Remote Technology Corporation, Oak Ridge, Tennessee. Phase I of the contract resulted in a determination that the use of remotely controlled survey and inspection devices was feasible for cost effective applications in certain high dose areas of nuclear power plants.

Phase II has resulted in the fabrication (from commercially available components) of a tethered survey and inspection system (SURBOT) scheduled to be demonstrated in an operating power plant in FY 86. The Remotec robot is capable of: taking radiation measurements; air sampling; surface contamination sampling; temperature, humidity and sound measurements; and high-resolution viewing of critical components. In addition the system can be programmed to perform a variety of tasks without operator intervention. The system has just completed extensive testing by the Electric Power Research Institute (EPRI) which was enthusiastic about its performance.

SURBOT will be demonstrated continuously for the NRC staff and others on Wednesday, December 11, 1985, in Room 118 of the Phillips Building from 12:00p.m. to 4:30p.m. This will be an informal program and attendance for any half-hour during the afternoon should be adaquate to see the range of capabilities of SURBOT. Attendees will have an opportunity to operate the system.

International Workshops on Dosimetry of Beta Particles and Low-Energy X-Rays and Radiation Damage to Skin

Two workshops, co-sponsored by the U.S. Department of Energy and EURADOS, the European Radiation Dosimetry Group, were held recently at the Commissariat a l'Energie Atomic (CEA) in Saclay, France. The first workshop, Dosimetry of Beta Particles and Low-Energy X-rays, was held October 7-9, 1985; the second, Radiation Damage to Skin, Fundamental and Practical Aspects, was held October 9-11, 1985. Several important conclusions that could affect radiation protection in the U.S.A. were reached by the participants. They reaffirmed that the dose from low-energy x-rays and from beta particles should continue to be measured at a depth in tissue of 70 um as recommended by the International Commission on Radiation Protection (ICRP-26) and as required by current NRC regulations. Commercially-available personnel dosimeters are not suitable for direct dose measurement at a tissue depth of 70 um, but the dose at this depth can be estimated by extrapolation if a multi-element dosimeter is used. Since

many commonly-used dosimeters do not use enough elements for this extrapolation, estimates at greater depths are usually accepted. Another consensus of the group was that ultra-thin thermoluminescent dosimeters, when fully adaptable for use in the workplace, will replace the multi-element dosimeters now commonly used for personnel dosimetry of beta particles and low-energy x-rays. From other information presented it appears that the minimum dose measurable using thermoluminescent dosimeters may be lowered considerably by using computer analysis techniques with current dosimeter readout equipment. The workshop participants exhibited special interest in a portable beta spectrometer whose performance is currently being evaluated in field tests conducted for the NRC by the Idaho National Engineering Laboratory.

Contact:
R. B. Neel
42-74559

RES Rulemaking Activities

Severe Accident Risk Assessment

There are several important highlights from last week's activities, summarized below:

1. Overall responsibility for the Zion analysis was assigned to BNL, with SNL providing an accident sequence analysis in a letter report and BCL providing source term code runs.

2. Agreement was reached by all parties on the approach to sensitivity analyses, and a final letter of instruction will be sent to SNL within a week.

3. Schedules for all plants were completely reviewed and revised and presented in the November 4 Commission briefing (ref. Attachment) While slips of about one to two months have been typical, these were expected due to the very optimistic nature of original schedule. The schedule for the public draft of NUREG-1150 is unchanged --mid summer of 1986.

4. A kickoff meeting on uncertainty white papers will be held with all project managers on November 7. This work will be closely coordinated with SNL and BNL so as to result in common assumptions and ranges of parameters to the extent possible in NUREG-1150. The work will follow the guidelines established in 2, above.

Shipping Cask Response to Severe Transportation Accidents-Modal Study

The purpose of this study is to determine the degree of safety being provided during shipments of spent fuel. This purpose is being accomplished by assessing the performance of licensed casks when subjected to severe transportation accident environments. A final report is scheduled to be completed by mid-December.

In a discussion with F. Gillespie, G. Cummings (LLNL) indicated that LLNL is adhering to the schedule for submission of specific sections of the Engineering Document and supporting Appendix material. The introductory section and the outline for the analysis and results sections were received November 4. Dir RES and D Dir NMSS will be visiting LLNL on November 14 for an overall program review and to review progress on the final report documentation.

Cost Benefit Workshops

Past Commission and EDO policy and planning guidance suggest "... existing guidance on performing cost-benefit analysis should be further developed." In accordance with this guidance, RES under a technical assistance contract with PNL has completed a series of cost-benefit workshops at all the Regions to support current policies and procedures affecting management of plant-specific backfitting of nuclear power plants.

ENCLOSURE E

The workshop had two objectives: (1) to identify recognized guidance documents within the NRC to support performance of regulatory analysis which must accompany all backfits proposed, and (2) to train the staff in the application of existing cost-benefit methods to backfits including resources available to assist in performing high quality regulatory analyses.

The workshops began in late July 1985 and ended November 4, 1985.

NOV 8 1985

10/31/85
(REVISED)

FINAL SARRP SCHEDULE FOR NUREG-1150 AND NRR SUPPORT

	ACTIVITY	SURRY	PEACH BOTTOM	SEQUOYAH	GRAND GULF	ZION*	LASALLE
1.	ACCIDENT SEQUENCE INITIAL INPUT	C	C	C	C	C	12/19
2.	SOURCE TERM BINNING	C	C	C	C	C	1/23
3.	NUMBER OF SOURCE TERM CODE RUNS	~3	6	~8	3	~6	~6
4.	SOURCE TERM CODE RUNS	C (EXCEPT 1)	C	11/15	11/15	12/31	3/20/86
5.	RELEASE CHARACTER-ISTICS	C (EXCEPT 1)	C	11/22	11/25	1/15/86	4/14/86
6.	CONSEQUENCE CALC.	C (EXCEPT 1)	11/15	12/10	12/11	1/31/86	4/28/86
7.	REFINE ACCIDENT SEQUENCES	C	11/11	11/15	12/13	N/A	2/27/86
8.	CONTAINMENT TREES DRAFT REPORT	11/15	12/9	12/3	1/20/86	1/31/86	5/28/86
9.	BASELINE RISK CALCULATION	11/19	11/25	12/18	1/7/86	2/15/86	5/26/86
10.	RISK/RISK RED. TABLES	12/9	1/20/86	1/21/86	3/14/86**	3/28/86**	6/9/86**
11.	RISK/RISK RED. DRAFT DETAILED RPT.	2/26/86**	4/6/86**	4/1/86**	4/15/86	4/30/86	7/7/86
12.	RISK/RISK RED. FINAL DETAILED RPT.	4/23/86	6/1/86	5/27/86	6/10/86	6/30/86	9/1/86

*TENTATIVE SCHEDULE

**INCLUDES FINAL SENSITIVITY ANALYSES AND UNCERTAINTY RANGES

ENCLOSURE 3

ATTACHMENT TO RES RULEMAKING ACTIVITIES

Publications to be Issued in the Near Future

Title: Instrument Setpoints for Safety-Related Systems
(R. G. 1.105, Rev. 2)

Description: This guide describes a method acceptable to the NRC staff for complying with the Commission's regulations for ensuring that instrument setpoints are initially within and remain within the technical specification limits.

Contact: A. S. Hintze
443-7712

Title: Radiation Damage to Reactor Vessel Materials
(Draft R. G. 1.99, Proposed Rev. 2)

Description: Provides calculative procedures and other guidance for accounting for radiation damage in regulating fracture prevention of reactor vessels.

Contact: Pryor N. Randall
443-7711 or 492-8186

ENCLOSURE E

NOV 8 1985

ITEMS OF INTEREST
OFFICE OF INTERNATIONAL PROGRAMS
WEEK ENDING NOVEMBER 8, 1985

International Visitors

On Monday and Tuesday Mr. Michel Laverie, Deputy Chief of the French Service Central de Surete des Installations Nucleaires (SCSIN), accompanied by three SCSIN members, met with NRR, RES, NMSS and IP staff for technical discussions. Technical topics covered included steam generator tubes, station blackout and decommissioning.

On Monday and Tuesday Mr. B. John Darlaston of the UK Central Electricity Generating Board's Berkeley Nuclear Laboratories met representatives of the NRR Divisions of Engineering and Systems Integration and the RES Divisions of Engineering Technology and Risk Analysis and Operations to discuss various issues related to material and system integrity.

On Tuesday Messrs. Ishibashi and Takahashi, lawyers of the Committee of Environmental Pollution Prevention of Japan's Federation of Bar Associations, visited NRC. The visitors are studying the potential health hazards of the proposed Japanese reprocessing and waste storage facility in the Aomori Prefecture. They met with ELD, RES and NMSS to gather information on U.S. experience and plans regarding nuclear waste management and reprocessing.

On Thursday Messrs A. Pugliesi, T. Sano, and L. Lojello of Italy's ENEA/DISP visited NRR to discuss recent developments in geotechnology and site specific seismicity.

On Friday a five-member technical team from China's Ministry of Nuclear Energy, Radiometrology Center, met with staff members from NMSS, RES, and IP for a discussion of NRC's regulations and radiometrology. The purpose of the team's visit to the U.S., which is being hosted by the National Bureau of Standards, is to learn about the present status and future development of research work in the U.S., with emphasis on neutron standards, radionuclides standardization, and radiation dosimetry standards.

Foreign Trip Reports

Guy H. Cunningham III, Executive Legal Director
September 29-October 2, 1985; Visited the FRG:

Mr. Cunningham attended the International Nuclear Law Association Congress, Nuclear Inter-Jura 1985, in Constance. As Chairman of the Association's working group on nuclear third party liability issues, he presented the report of the working group's activities of the past two years.

ENCLOSURE G

NOV 8 1985

Foreign Trip Reports continued

Shlomo S. Yaniv, HEBR/RES
September 30-October 2, 1985; Visited the FRG:

Mr. Yaniv attended the Workshop on Radiological Health Effects Models for Nuclear Accident Consequence Analysis at the Karlsruhe Nuclear Research Center. Its objective was the evaluation of the health effects models and risk coefficients proposed in NUREG/CR-4214, published by Sandia National Laboratory, under contract to NRC.

Themis P. Speis, Director, DST/NRR (Head of NRC Delegation)
September 30-October 4, 1985; Visited France:

The purpose of the delegation's trip was to discuss NRC's initial draft report on the design differences between the French P4 1300 MWe plant and a typical U.S. four-loop PWR. The participants visited the Paluel nuclear power plant on October 1.

ENCLOSURE G

OFFICE OF STATE PROGRAMS

ITEMS OF INTEREST

WEEK ENDING NOVEMBER 8, 1985

MAINE INITIATIVE ON LOW-LEVEL WASTE

According to United Press International reports, Maine voters approved an initiative which would allow them the right to vote for or against any plan for the storage or disposal of Low-Level Waste in the State. UPI claims that to avoid a run-off, the vote must be at least 50%. The measure in this instance reportedly won by 50.2%. These figures are based solely on UPI reports (the only ones available at this time), and as such are unofficial.

ENCLOSURE H

NOV 8 1985

OFFICE OF RESOURCE MANAGEMENT

Item Of Interest

Week Ending November 8, 1985

Energy and Water Development Appropriations Act

The President signed the Energy and Water Development Appropriations Act of FY 1986 (Public Law 99-141) on November 1, 1985. This Act appropriates $418 million to NRC for FY 1986.

ENCLOSURE I

NOV 8 1985

REGION I

ENCLOSURE

TMI-1 STATUS REPORT FOR THE PERIOD NOVEMBER 1-8, 1985

1. Plant Status

As of 8:00 a.m. on November 8, 1985, TMI-1 was at 48% power.

2. Test Program Status

The licensee's planned test program and current status for restart of TMI-1 are shown on the attached Figure 1. Special testing planned for the afternoon on November 8, 1985 includes an integrated control system (ICS) tuning test that requires a change in power from 48% to 40%, then back to 48%. This test was originally planned for the week of October 27, 1985, but was rescheduled, pending completion of ICS system troubleshooting and maintenance on a feedwater pump controller to permit the test to be performed with both of the main feedwater pumps in automatic.

3. Facility Operations Summary

Steady-state operation at 48% power continued throughout the period.

4. Items of Special Interest

NRC Notifications

As has been the case in prior periods, there were no events that required notification of NRC by the licensee. There were, however, several events of interest and they are discussed below.

Moisture Separator Drain System Steam Leak

On Saturday, November 2, 1985, at 9:30 a.m., the licensee attempted to repair a small water leak on a flange in the secondary plant moisture separator drain system which is located in the turbine building. Because the section of piping was not effectively isolated, a steam leak occurred at the flange as the flange bolts were loosened. The leak was fed by relatively hot water from the steam and feedwater system. Some of the workers involved in the repair received superficial burns that did not require offsite medical attention. The steam leak lasted approximately 20 minutes. Although a minor perturbation was noticed in feedwater flow, the plant remained at steady-state 48% power conditions. This matter is reported for information and is not a significant NRC concern.

Loss of Alternate Power Source for ICS/NNI Bus

On November 5, 1985, a 120 volt alternating current bus was deenergized by licensee technicians who opened a supply breaker in response to arcing

and sparking at an adjacent breaker. The technicians were performing breaker preventive maintenance on an engineered safeguards bus at the time. Although no significant loads were lost by the bus being deenergized, the electrical incident was apparently due to technician error and is of interest because it had the potential for causing a plant transient.

The deenergized bus was an alternate power source for the integrated control system (ICS)/non-nuclear instrumentation (NNI) bus. The ICS and NNI remained energized. The reactor protection and engineered safeguard actuation systems were unaffected because they were powered from separate vital buses. The licensee has taken disciplinary action against the technicians. Additional licensee corrective action will be reviewed by the TMI-1 Restart Staff.

Letdown Cooler Leakage

On November 5, 1985, the licensee removed one of the two letdown coolers from service because of an indicated leakage at the rate of 1.3 liters/day. The letdown cooler is part of the makeup and purification system and is used to lower the temperature of the reactor coolant such that it can be processed and then returned to the reactor coolant system. Leakage of the reactor coolant was into the intermediate closed cycle cooling water system, a non-safety system. The leakage was first detected on October 24, 1985, by radioactive noble gas sampling and trended on a daily basis. Although the cooler is still considered to be operable by the licensee, it is not in service. The licensee has two spare letdown coolers available onsite if needed. This matter is an item of interest to the TMI-1 Restart Staff because of the leakage of reactor coolant.

Emergency Drill

The licensee is planning a practice emergency drill on November 13, 1985, in preparation for the annual emergency exercise on November 20, 1985. The drill was originally scheduled for November 6, 1985. The TMI-1 Restart Staff, in conjunction with NRC TMI-2 Program Office personnel, plans to participate in the practice drill for a limited time to exercise the NRC site emergency plan.

5. NRC Thermoluminescent Dosimeter (TLD) Special Monitoring Program for TMI

The special NRC TLDs for TMI for the period October 18 - October 31, 1985, were processed at the Region I TLD laboratory. The TLD readings are provided in the attached Table A. The monitoring results indicate that the radiation levels at these monitoring locations remain at natural background levels. These readings can be expected to vary slightly from period to period due to variations in natural background, independent of releases from the plants.

6. Meeting with Members of TMI Alert

On November 11, 1985, the TMI-1 Restart Staff Director and Restart Manager met with three members of TMI Alert (TMIA). The primary purpose of the meeting was to discuss the usefulness of the TMI-1 Restart Staff's weekly status reports from TMI Alert's perspective. Their general conclusions were that the reports were of little value in that they contained insufficient detail and were received too late. We informed them that we were attempting to improve both of these areas. We also discussed and answered questions regarding recent activities at TMI-1 addressed in the weekly status reports. Some of the requests for more technical information could not be addressed at the meeting and will be provided in a letter.

The role of TMIA also was discussed. During our conversation TMIA representatives indicated that they now perform a community service by responding to questions and concerns by the local public. Their independent review of GPUN local environmental (background) radiation data is available through the local emergency management agency office.

The TMIA representatives expressed their view that they represent the majority of local opinion which they believe was reflected in the local referendum (against the restart of TMI-1) a number of years ago. They expressed their dismay that federal agencies (primarily NRC) repeatedly have ruled in favor of the licensee ever since the first significant decision since the TMI-2 accident -- the venting of krypton from the TMI-2 reactor building. Consequently, they continue to believe these same federal agencies are not really concerned with local public health and safety.

7. TMI-1 Restart Staff Status During the Period

The TMI-1 Restart Staff continued 16-hour shift coverage during the period. The shifts were manned by NRC personnel from Region II and the Reactor Training Center, and by a reactor operator examiner from EG&G Idaho, Inc., an NRC contractor. Two Region I reactor engineers and a senior radiation specialist were onsite during portions of the period to augment the resident inspection staff.

The staff's inspection plan for this period covered the primary functional areas of operations, maintenance, surveillance, and radiation protection with the division of responsibility as noted in previous status reports. The TMI-1 Restart Staff continued to evaluate the performance of licensee personnel and the plant to determine whether the licensee should be permitted to proceed beyond the next hold point, which is reached after 30 days of operation at 48% power.

Contact from the public was minimal throughout the period. We continued to maintain daily contact with representatives of the Commonwealth of Pennsylvania during this period.

The TMI-1 Restart Staff issued two daily highlight reports for the Executive Director for Operations on November 4 and 6, 1985, regarding the moisture separator drain system leak, the 120V ac bus deenergization, and the letdown cooler leakage. The fourth weekly status report for the period October 25-November 1, 1985, was issued on November 1, 1985. Inspection Report No. 50-289/85-24, for the period October 11-18, 1985, was issued on November 7, 1985. The findings were generally favorable, however, we noted an adverse trend in the supervisory oversight of personnel working in safety-related areas. This has resulted in instances of personnel standing on safety equipment and a potential violation regarding scaffolding that was not properly secured. We noted that although no adverse effect on the plant has yet resulted, the licensee's prompt attention is needed to reverse this trend.

8. TMI-1 Restart Staff Composition During Period

The TMI-1 Restart Staff was comprised of the following personnel during the period:

W. F. Kane, TMI-1 Restart Director
R. J. Conte, TMI-1 Restart Manager
D. R. Haverkamp, Technical Assistant
F. I. Young, Resident Inspector, TMI-1
D. M. Johnson, Reactor Engineer
R. J. Urban, Reactor Engineer
J. R. White, Senior Radiation Specialist
T. L. Morgan, Shift Inspector, EG&G Idaho, Inc.
L. J. Reidinger, Shift Inspector, Reactor Training Center
T. F. Stetka, Shift Inspector, Region II
C. P. Hix, Secretary
L. M. Prough, Secretary

TABLE A

TMI SPECIAL TLD MONITORING RESULTS

Station	Distance (miles)	Direction	Baseline mR/day (Mean ± s.d.)	Field Exposure 10/18/85-10/31/85 mR/day (Mean ± s.d.; total uncertainty)
	Control		-	0.17 ± 0.0007; 0.03
2	3.9	101	0.19 ± 0.02	0.20 ± 0.01; 0.03
3	2.7	109	0.16 ± 0.02	0.16 ± 0.01; 0.02
4	1.8	163	0.16 ± 0.02	0.18 ± 0.01; 0.03
5	2.2	161	0.18 ± 0.02	0.16 ± 0.003; 0.02
6	1.0	150	0.17 ± 0.03	0.18 ± 0.005; 0.03
7	0.6	136	0.17 ± 0.02	0.16 ± 0.001; 0.02
8	0.4	83	0.16 ± 0.03	0.17 ± 0.02; 0.03
9	0.5	60	0.16 ± 0.02	0.18 ± 0.001; 0.03
10	1.7	1	0.14 ± 0.02	TLD MISSING
11	0.9	25	0.16 ± 0.01	0.17 ± 0.003; 0.03
12	2.8	46	0.16 ± 0.02	0.18 ± 0.004; 0.03
14	2.5	358	0.14 ± 0.02	0.18 ± 0.01 0.03
16	3.1	0	0.14 ± 0.02	0.17 ± 0.004; 0.03
18	3.5	349	0.17 ± 0.03	0.15 ± 0.01; 0.02
19	3.2	343	0.17 ± 0.02	0.19 ± 0.004; 0.03
20	5.0	318	0.16 ± 0.01	0.16 ± 0.004; 0.02
21	1.3	348	0.13 ± 0.01	0.15 ± 0.01; 0.02
22	3.1	17	0.17 ± 0.02	0.18 ± 0.03; 0.03
23	3.8	64	0.13 ± 0.01	0.17 ± 0.01; 0.03
24	3.6	44	0.17 ± 0.01	0.17 ± 0.01; 0.03
	Control		-	0.14 ± 0.001; 0.02
34	2.3	267	0.17 ± 0.01	0.18 ± 0.02; 0.03
35	1.8	299	0.17 ± 0.01	TLD MISSING
36	1.2	267	0.12 ± 0.02	0.16 ± 0.02; 0.02
37	1.4	256	0.14 ± 0.01	0.16 ± 0.01; 0.02
38	1.9	225	0.18 ± 0.02	0.17 ± 0.003; 0.03
39	2.1	200	0.13 ± 0.01	0.16 ± 0.01; 0.02
40	2.5	204	0.16 ± 0.02	0.17 ± 0.01; 0.03
46	3.0	177	0.14 ± 0.02	0.17 ± 0.001; 0.03
50	4.9	145	0.14 ± 0.04	0.16 ± 0.001; 0.02

Abbreviations:
mR = millirem
s.d. = standard deviation

REMARKS:
Twenty-nine environmental (offsite) locations are monitored on a two-week exchange cycle using special TLDs for the TMI site. Two control TLDs were stored in a 1/2" thick lead shield at the NRC TMI Office for two weeks. The baseline data for the environmental monitoring were calculated using the past ten quarters of monitoring results. To date, all monitoring results indicate normal natural background radiation levels.

ENCLOSURE L

NOV 8 1985

REGION III
STATUS REPORT

DAVIS-BESSE LOSS OF FEEDWATER EVENT
NOVEMBER 8, 1985

Plant Status

The plant remains in cold shutdown. The circulating water canal is drained. Decay heat removal loop No. 1 is filled and being tested in preparation for taking decay heat loop No. 2 out of service for maintenance. Emergency Diesel Generator (EDG) No. 1 is now operable. Both EDGs have a control system design deficiency that renders them inoperable throughout the ten minute period that they run at idle speed following normal shutdown. While running at idle speed the output circuit breaker will open after closing due to an undervoltage condition on the associated bus. The licensee has taken appropriate actions to compensate for and correct this condition, which was discovered during the licensee's review of IE Notice 85-73. The diesel firewater pump is now operable.

ENCLOSURE L

NOV 8 1985

Region III STAFF ACTIONS AS ASSIGNED BY W. DIRCKS MEMO OF AUGUST 5, 1985

NOV 8 1985

1. Item: Adequacy of the licensee's management and maintenance activities

ACTION	STATUS/COMMENTS
(b) Evaluate and take action on the licensee's response to findings concerning management practices (e.g., control of maintenance programs and post-trip reviews).	Region III is observing maintenance activities and evaluating LER's, DVR's and other items as they relate to the control of maintenance. A maintenance survey team inspection conducted on September 16-20, 1985 confirmed previous concerns identified by Region III. A followup team inspection is planned for early 1986. DRP in conjunction will DRS will assess management practices prior to restart.

6. Item: Reliability of the AFW containment isolation valves and other safety-related valves

ACTION	STATUS/COMMENTS
(a) Monitor the licensee's trouble-shooting activities	Troubleshooting activities related to containment isolation valves AF599 and AF608 are complete. Region III specialists are evaluating and monitoring MOVATS testing on other safety-related motor operated valves. About 75% of 166 valves have been completed.
(e) Determine that the procedures for adjustments of the AFW isolation valves such as torque switch bypass switches are clear and proper, and that associated training programs are adequate. Confirm that adjustment settings are consistent with plant procedures.	Limitorque operators for valves AF599 and AF608 have been adjusted and the valves have been tested under dp conditions. Confirmatory testing will be performed during startup.

ENCLOSURE 1

NOV 8 1985

7. Item: Adequacy of emergency notifications

ACTION	STATUS/COMMENTS
(a) Verify the adequacy of the licensee's procedures and training for the reporting of events to the NRC Operations Center.	Investigation of the adequacy of the licensee's procedures and training for the reporting of events to the NRC Operations Center has been completed. One violation of NRC requirements was found. Inspection results are documented in Inspection Reports 85023 and 85034.

8. Item: Reliability of AFW pump turbines

ACTION	STATUS/COMMENTS
(a) Monitor the licensee's troubleshooting activities including possible hot plant operation to confirm failure mode.	Troubleshooting activities are complete. Region III will monitor confirmatory testing during plant startup as part of startup test activities.
(d) Verify that the AFW system has been adequately tested to confirm system configuration involved with design basis events.	Results of test review group meetings have culminated in the development of a charter outlining test review group responsibilities and activities. AFW testing will be included as part of the testing review team effort.
(e) Review the implementation of the operator training program to assure proper operator actions, such as resetting of trip throttle valve.	The licensee is developing a training program on resetting of the trip throttle valve. This training will include resetting of trip mechanism at operating conditions, therefore, the training will be completed during plant restart.

9. Item: Reliability of the PORV

ACTION	STATUS/COMMENTS
(a) Monitor the licensee's troubleshooting activities	Troubleshooting activities have been completed. Confirmatory testing will be conducted at operating temperature and pressure. Testing of an equivalent valve indicated a lower flow condition than expected. Flow results from Crosby (valve manufacturer) testing were higher than the flow results from the Marshall Steam Station test facility. All test data indicated flow was less than expected. The licensee will continue to evaluate this information.

ENCLOSURE 1

NOV 8 1985

12. Item: Resolution of equipment deficiencies

ACTION	STATUS/COMMENTS
(a) Monitor the licensee's trouble-shooting activities	All the Region III activities relative to troubleshooting/root cause activities have been completed and are documented in Inspection Reports 85-021, 85-022 and 85-025.*

ENCLOSURE 1

*Inspection Report 85025 is in draft and will be issued in approximately 1 week.

Other Activities

All work on the new electric motor driven startup feedwater pump (SUFP) should be completed by early next week. Calibration of the associated instrumentation and testing of the system should also begin early next week.

Evaluations and inspections of 2155 safety system piping supports are continuing. Nonconformance Reports (NCRs) have been written against 1594 of 1770 pipe supports inspected so far. Rework is required on 114 of the 872 supports that have been evaluated by the licensee's engineering department. Rework has been completed on 23 supports. There are 16 licensee inspection teams working on this project.

MOVATS testing continues on the 166 safety-related motor-operated valves onsite. Of the 120 valves that have been tested, only a few will require retesting. The licensee is now repairing and testing valve motor operators continuously with three overlapping ten-hour shifts.

The licensee has performed engineering evaluations of the three hydraulic snubbers that did not meet the Technical Specification required Surveillance Test acceptance criteria. The evaluation revealed that although the snubbers required repair or adjustment they were operable. Therefore testing of additional snubbers is not required.

The NRC test review group has completed the preliminary review of the licensee's SRTP with satisfactory results. The draft Technical Evaluation Report (TER) will be sent to NRR next week. A final TER is anticipated by the end of November, subject to the licensee's schedule. The review team will continue to provide weekly coverage of licensee testing activities. The licensee's schedules show this effort complete by February 2, 1986. The test review group has assessed the licensee's schedule and concluded that completion in early March, 1986, is more realistic.

ITEMS ADDRESSED BY THE COMMISSION - WEEK ENDING NOVEMBER 8, 1985

A. STAFF REQUIREMENTS - BRIEFING ON STATUS OF SAFETY GOAL EVALUATION REPORT, 2:00 P.M., TUESDAY, OCTOBER 22, 1985, COMMISSIONERS' CONFERENCE ROOM, D.C. OFFICE (OPEN TO PUBLIC ATTENDANCE) Memo SECY to W. J. Dircks dated 11/5/85

The Commission met to discuss the status of the report "Safety Goals for Nuclear Power Plants," NUREG-0880, which was issued in 1982.

Commissioner Zech requested information as to why the staff has modified its position on averted onsite costs and the core melt design objective in the safety goal report.
(EDO) (SECY Suspense: 11/15/85)

The Commission requested staff to provide the Commission with a list of questions or open issues to which the staff requires answers or guidance from the Commission in order to proceed with work on the safety goal report.
(EDO) (SECY Suspense: 11/15/85)

The Commission indicated that it would schedule a meeting in the near future with the Advisory Committee on Reactor Safeguards (ACRS) to discuss the safety goal report.

(Subsequently, the Commission scheduled a meeting to discuss the safety goal report and other items with the ACRS on Thursday, November 7 at 2:00 p.m.)

B. STAFF REQUIREMENTS - STATUS OF INTERPRETATION OF APPENDIX R - FIRE PROTECTION, 2:00 P.M., THURSDAY, OCTOBER 3, 1985, COMMISSIONERS' CONFERENCE ROOM, D.C. OFFICE (OPEN TO PUBLIC ATTENDANCE) Memo SECY to W. J. Dircks and H. H. E. Plaine dated 11/6/85

The Commission met with staff to discuss the status of the implementation of fire protection requirements, the fire protection rule contained in 10 CFR 50.48 and Appendix R and the staff's recommendations regarding implementation of Appendix R contained in SECY-85-306.

The Commission requested a report from the staff on current fire protection enforcement actions as well as a suggested schedule for affected licensees to comply with Appendix R requirements.
(IE/NRR) (SECY Suspense: 11/29/85)

B. CONTINUED

Chairman Palladino urged the Commissioners to vote promptly on SECY-85-306. Additionally, he would like to see the staff incorporate a license condition in each facility license to ensure enforceability of fire protection programs. This should be done in addition to incorporating fire protection programs into the next edition of the FSARs. The license condition should allow flexibility which permits modifications of fire protection programs in accordance with 10 CFR 50.59.

Commissioner Asselstine requested that OGC provide its comments on staff's reply to the Trubatch memorandum on Appendix R requirements.
(OGC)

(Subsequently, on October 24, 1985 the General Counsel provided a response to the Commission.)

Commissioner Asselstine asked the staff to send to him the cost-benefit analysis required by the backfit rule of the new or different interpretations of Appendix R. If such cost-benefit analysis does not now exist, he asked that the required backfit analysis be performed and sent to him before any new or modified interpretations are transmitted to licensees.
(NRR) (SECY Suspense: 11/29/85)

C. STAFF REQUIREMENTS - DISCUSSION OF FITNESS FOR DUTY, 10:00 A.M., TUESDAY, OCTOBER 22, 1985, COMMISSIONERS' CONFERENCE ROOM, D.C. OFFICE (OPEN TO PUBLIC ATTENDANCE) Memo SECY to W. Dircks dated 11/8/85

The Commission met with the staff to discuss fitness for duty for plant personnel in the nuclear industry.*

The Commission directed the staff to revise the policy statement on fitness for duty to include broad objectives: the expectation that each utility will include provisions for chemical testing in monitoring programs along with the expectation that each utility will develop a disciplinary program consistent with the EEI guidelines. This policy statement should cover plant operating personnel as well as construction workers performing work in safety related areas.
(IE) (SECY Suspense: 11/21/85)

The Commission requested the staff to provide an analysis of the consequences to the health and safety of the public of having personnel unfit for duty at NRC licensed facilities other than power reactor facilities (i.e., fuel cycle facilities and research reactors) and discuss the role of NRC, if any, in each case.
(IE/NMSS) (SECY Suspense: 1/6/86)

C. CONTINUED

Commissioner Bernthal requested that the staff provide a list of utilities that have chemical screening programs and those that do not.

(IE) (SECY Suspense: 11/15/85)

* The Commission has under consideration a memorandum from Commissioner Bernthal dated October 31, 1985, subject "Fitness for Duty Policy Statement" (COMFB-85-16), a copy of which was provided to the staff.

NOV 8 1985

NRR MEETING NOTICES*

NOVEMBER 8, 1985

DATE/TIME	DOCKET NUMBER	LOCATION	PURPOSE	APPLICANT/ ATTENDEES	NRR CONTACT
11/12/85 10:00 am		P-110 Bethesda	To discuss auxiliary actuating devices for PWR Safety Valves which have been proposed for use in France	Vortech	F. Cherny
11/12-15/85	50-443 50-444	PSNH Seabrook Off. 7910 Woodmont Ave. Bethesda, Md.	To discuss technical specification review of Seabrook's proposed tech specs	Public Serv. of New Hampshire	V. Nerses
11/13-14/85 9:00 am	50-247	Reg. I Office King of Prussia, PA	To discuss Inservice Testing Program for Indian Point 2	Consolidated Edison	M. Slossen
11/14-15/85 9:00 am	50-315 50-316	Donald Cook Site Stevensville, Mich.	To discuss wrong unit/wrong train events at Cook for use in developing industry guidance for prevention	Indiana and Michigan Elec. Co.	D. Wigginton
11/15/85 9:00 am	50-280 50-281	P-114 Bethesda	To discuss Appendix R Exemption requests	Virginia Electric & Power Co. (Surry)	T. Chan
11/18/85 1:00 pm	50-331	P-110 Bethesda	Effect of leakage from purge/ vent valve inflatable seals on containment oxygen concentration	Iowa Elec. Light & Power Co.	M. Thadani

* Copies of summaries of these meetings will be made publicly available and placed in the respective docket file(s) in the NRC and local public document rooms

ENCLOSURE P

NOV 8 1985

NRR MEETING NOTICES*

DATE/TIME	DOCKET NUMBER	LOCATION	PURPOSE	APPLICANT/ ATTENDEES	NRR CONTACT
11/18-19/85	50-245	Millstone Site Waterford, Conn.	Review M-1 IPSAR, ISAP, and FTOL-SER Conversion	ACRS Northeast Nuclear Energy Co.	J. Shea
11/19-20/85 9:00 am	50-445 50-446	P-422 Bethesda	To discuss Supplemental Safety Evaluation Report Nos. 7-11	Citizens Assn. for Sound Energy	A. Vietti-Cook
11/20/85 9:30 am	50-219	P-114 Bethesda	To discuss long-range planning to complete deferments from Cycle 11R outage in the operating Cycle 11 - Oyster Creek	GPU Nuclear Corp.	J. Donohew
11/21/85 3:00 pm	50-400	Shearon Harris Site	Environmental qualification audit	Carolina Power & Light Co.	B. Buckley

Copies of summaries of these meetings will be made publicly available and placed in the respective docket file(s) in the NRC and local public document rooms.

ENCLOSURE 2

NMSS MEETING NOTICES

FOR WEEK ENDING: 11/8/85

Division of Fuel Cycle and Material Safety

NOV 8 1985

DATE/TIME	DOCKET NUMBER	LOCATION	PURPOSE	ATTENDEES/ APPLICANT	NRC CONTACT
11/4-15/85	70-1257	San Francisco, CA	To attend ANS meeting.	G. Bidinger (FC)	Bidinger
		Richland, WA	Discuss dry UF_6 Process and Accompany Inspector at Exxon.	G. Bidinger (FC) RV Inspector Exxon Reps	
11/6/85	70-143	Atlanta, GA	To meet with Region II staff and discuss NFS license renewal application.	W. Crow (FC) A. Soong (FC) RII staff	Crow
11/12/85	71-9202	Willste Room 106	Discuss review of spent fuel cask for use at West Valley, New York.	C. MacDonald (FC) R. Odegaarden (FC) R. Chappell (FC) K. Goldmann (Transnuclear) Reps of EG&G, DOE & ORNL	MacDonald
11/18/85	72-3	Willste 5th floor conf room	To discuss Carolina Power & Light Co. (CP&L) responses to NRC questions regarding CP&L dry storage application for the H.B. Robinson site.	D. Koss, et al. (CP&L) J. Roberts (FC) F. Sturz (FC) J. Schneider (FC)	Roberts
11/18-20/85	Project M-32	West Valley, NY	Visit to West Valley to discuss QA responses, LLW form test data and SAR review.	T. Clark (FC) T. Johnson (WM) R. Person (WM) Reps of West Valley	Clark
11/18-21/85		Atlanta, GA	To attend two DOE meetings: Fall Info. Meeting of the Office of Civilian Radioactive Waste Management and a workshop to discuss planning for transportation under NWPA.	J. Cook (FC) W. Thompson (FC) Reps of DOE	Cook

ENCLOSURE P

NOV 8 1985

NMSS MEETING NOTICES

FOR WEEK ENDING: 11/8/85.

Division of Fuel Cycle and Material Safety

DATE/TIME	DOCKET NUMBER	LOCATION	PURPOSE	ATTENDEES/ APPLICANT	NRC CONTACT
11/19/85 8:30 am		Baltimore, MD	Presentation at Johns Hopkins University - Seminar on Low-Level Waste - Under the auspices of CARER.	R. Cunningham (FC)	Cunningham

Division of Safeguards

None

Division of Waste Management

November 18-19	Atlanta, GA	Information Management Meeting (Fall) with DOE/Office of Civilian Radioactive Waste Management	NStill DOE staff	NStill

ENCLOSURE P

NOV 8 1985

RES MEETING NOTICES

November 8, 1985

DATE/TIME	DOCKET NUMBER	LOCATION	PURPOSE	ATTENDEES/ APPLICANT	NRC CONTACT
11/13-14/85		Sandia Lab., Albuquerque,NM	A Conference on Security Vehicle	Ten Eyck,NMSS Ting,RES	Ting
11/14/85		LANL, Los Alamos	Contract reviews (vital equipment/ area identification techniques & non-destructive reference book)	Ting, RES LANL personnel	Ting
11/18-22/85		CEA Hdqs Paris, France	HYDROCOIN workshop and coordination group meeting	International government agencies & contractors dealing w/ground-water modeling for HLW & LLW facilities	Nicholson
12/2-4/85 1/27-29/85		TBD	Review the needs for validation of mathematical models for waste repository licensing in workshop format	RES contractors and selected individuals by invitation	Randall

ENCLOSURE P

NOV 8 1985

OSP MEETING NOTICES

November 8, 1985

DATE/TIME	LOCATION	PURPOSE	ATTENDEES	NRC CONTACT
11/15/85	Bethesda, Md.	Meeting with Illinois to discuss Sec. 274b Agreement	Illinois-T. Lash and staff NRC-OSP, NMSS & ELD Staff	DNussbaumer
11/18/85	New York, NY	Attend National Association of Regulatory Utility Commissioners (NARUC) Annual Convention	PUC Representatives	JSaltzman
11/18-22/85	Austin, Texas	Technical Review Committee Meeting on the Suggested State Regulations	State and Federal Agency Representatives	LBolling
11/19/85	Farmington, CT	Discuss issues concerning Price-Anderson with American Nuclear Insurers (ANI)	ANI Representatives	JSaltzman

ENCLOSURE P

NOV 8 1985

AEOD MEETING NOTICES

DATE/TIME	LOCATION	PURPOSE	ATTENDEES	NRC CONTACT
11/19/85 9:00 a.m.	Maine Yankee Atomic Power Plant	To collect and discuss information concerning the instrument air supply to the temperature control valves in the component cooling system which supplies cooling to both Emergency Diesel Generators.	Stuart Rubin (AEOD) Eric Leeds (AEOD)	E. Leeds

ENCLOSURE P