

NRC Form 756
(9-83)U.S. Nuclear Regulatory Commission
Approved OMB No. 3150-0104

LICENSEE EVENT REPORT (LER)

Facility Name(1) Maine Yankee Atomic Power Company	Docket Number(2) 10 15 10 10 10 31 01 9	Page(3) 1 of 02
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Title(4)

Type A Test Failure Due To Integration Of Type C Test

Event Date(5)			LER Number(6)			Report Date(7)			Other Facilities Involved(8)		
Month	Day	Year	Sequential Number	Revision Number		Month	Day	Year	Facility Names	Docket Number(s)	
10	12	85	014	00		11	13	85		05000309	

This Report is Submitted Pursuant to the Requirements of 10 CFR §
(Check one or more of the following) (11)

Operating Mode (9)	1	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
Power Level (10)	10	20.405(a)(1)(i)	50.36(c)(1)	X 50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	Other (Specify in
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	Abstract below
		20.405(a)(1)(iv)	X 50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	and in Text, NRC
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	Form 366A)

LICENSEE CONTACT FOR THIS LER (12)

NAME Danny P. McDougald, Nuclear Safety Engineer	Telephone Number Area Code 2 0 7 8 8 2 6 3 2 1
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

Cause	System	Com- ponent	Manufac- turer	Reportable to NPRDS	Cause	System	Com- ponent	Manufac- turer	Reportable to NPRDS

Supplemental Report Expected (14) (If yes, complete Expected Submission Date)	Expected Month X	Expected Day 1	Expected Year 1
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On October 12, 1985, the Plant Engineering Department determined that the Type A test performed in accordance with 10CFR50 App. J had failed due to a Type C test failure on two primary component cooling valves which are inside and outside containment isolation barriers on the same penetration.

The leak rate of the two valves exceeded the limits of the local leak rate monitoring equipment. The unmeasurable leak rate, added to the Type A test, made the Type A test also unmeasurable.

Repairs were made during the 1985 refueling outage and the Type C test was repeated. As a result, the final Type A leakage rate is now less than the maximum allowable operational leakage rate.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

On October 12, 1985, the Plant Engineering Department determined that the Type A test required by 10CFR50 Appendix J had failed due to a Type C test failure on two primary Component Cooling (CC) Valves (V), inside and outside containment isolation barriers.

The primary CC piping to the Reactor Coolant Pumps (P) was not vented and drained during the performance of the Type A test because portions of the system were needed to cool the containment recirculation fans. Subsequent to the Type A test, additional valves and drains were added to this system to allow it to be drained and opened to containment during future Type A tests. Since the primary CC piping is inside the containment missile barrier and must be considered susceptible to failure from the impact loads of a postulated Loss of Coolant Accident, the Type C test local minimum pathway leakage rate of these two valves was added to the Type A test for correction.

The recorded leakage rate of the Type A test plus the addition of all required Type B and C tests for all systems that were not vented but should have been for the Type A test, except for the two primary CC Valves, was 0.0494 weight percent per day. This is well within the "as found" allowable leak rate of 0.0695 weight percent per day. The leak rate of the two primary CC Valves exceeded the limit of the local leak rate monitoring equipment. This unmeasurable leak rate, when added to the Type A test as required by Appendix J, made the Type A test also unmeasurable and in excess of the allowable overall integrated leakage rate.

Repairs were made to the two primary component cooling valves, as well as to other containment penetrations, during the 1985 refueling outage and Type C tests were repeated. When these test results were added to the Type A leakage, the resulting rate was then 0.0487 weight percent per day, which is less than the "as left" maximum allowable operational leakage rate of 0.0521 weight percent per day.

A detailed Summary Technical Report will be submitted to the NRC by Maine Yankee as required by 10CFR50 Appendix J, paragraph V.B.1.



ATOMIC POWER COMPANY •

EDISON DRIVE
AUGUSTA, MAINE 04336
(207) 623-3521

November 13, 1985
MN-85-191

GDW-85-281

Director, Office of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Document Control Desk

Reference: License No. DPR-36 (Docket 50-309)

Subject: Maine Yankee Licensee Event Report 85-014-00 - Type A Test Failure
Due to Integration of Type C Test

Gentlemen:

Please find enclosed Maine Yankee Licensee Event Report #85-014-00.
This report is submitted in accordance with the requirements of
10 CFR 50.73(a)(2)(ii, v).

Very truly yours,

MAINE YANKEE ATOMIC POWER COMPANY

G. D. Whittier, Manager
Nuclear Engineering and Licensing

GDW:bjp

Enclosure: Two pages

cc: Mr. Edward J. Butcher, Jr.
Dr. Thomas E. Murley
Mr. Cornelius F. Holden

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