

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Washington Nuclear Plant - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 9 7				PAGE (3) 1 OF 0 3											
TITLE (4) Reactor Building Ventilation Fire Damper Not Installed																									
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)												
1	0	1	1	8	5	8	5	0	5	7	0	0	1	1	0	8	8	5	0	5	0	0	0		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																							
POWER LEVEL (10)		20.402(b)				20.405(e)				50.73(a)(2)(iv)				73.71(b)											
		20.405(a)(1)(i)				50.38(e)(1)				50.73(a)(2)(v)				73.71(e)											
		20.405(a)(1)(ii)				50.38(e)(2)				50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)											
		20.405(a)(1)(iii)				X 50.73(a)(2)(i)				50.73(a)(2)(viii)(A)															
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)															
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)															
LICENSEE CONTACT FOR THIS LER (12)																									
NAME										TELEPHONE NUMBER															
W. S. Davison, Compliance Engineer										5 0 9 3 7 7 - 2 5 0 1															
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13): Ext. 2726																									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC															
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR											
YES (if yes, complete EXPECTED SUBMISSION DATE)										NO															

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On October 11, 1985, during performance of the fire damper inspection surveillance, it was discovered that ROA-FD-12 (Reactor Building Outside Air Fire Damper Number 12) had not been installed. The "B" RHR Heat Exchanger passageway area was immediately placed on the Fire Watch tour. Procurement of the fire damper was initiated. The inspection was continued to verify the presence of all fire dampers required by design. The area will remain on the Fire Watch tour until the damper is installed and is operational or until a temporary 3 hour fire barrier is installed in the duct as an interim corrective action.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Washington Nuclear Plant - Unit 2	0500039785	—	057	—	000	2	OF 03

TEXT (If more space is required, use additional NRC Form 386A-1 (17))

Plant Conditions

- a) Power Level - 72%
- b) Reactor Mode - 1

Event

On October 11, 1985, during the initial performance of Plant Surveillance Procedure PPM 7.4.7.7.2, "Reactor Building Exhaust HVAC Fire Rated Assembly (Fire Damper) Operation Inspection", it was discovered that ROA-FD-12 (Reactor Outside Air Fire Damper 12) had not been installed. ROA-FD-12 was designed as a 12"x 24" curtain type fire damper separating the "B" RHR (Residual Heat Removal) Heat Exchanger Room from the adjacent passageway. The fire damper was to have been mounted in a duct penetration through a concrete wall with the damper only being visible by removal of an air register face plate.

Investigation of this occurrence revealed the root cause as a cognitive personnel error by contractor and utility startup staff during the construction startup period. ROA-FD-12 appears to have been inadvertently eliminated from construction deficiency lists during a change of contractors. The fact that ROA-FD-12 was not installed was not identified during the construction turnover process by utility personnel. The personnel involved are no longer available on site for comment. All other fire dampers required by WNP-2 Technical Specifications were inspected and found to be installed, therefore the Supply System considers this incident an isolated singular occurrence.

Immediate Corrective Action

ROA-FD-12 was immediately placed on the 1 hour Fire Watch Tour in accordance with WNP-2 Technical Specifications.

Further Corrective Action

- o The Surveillance Procedure to inspect all fire dampers was continued until 100% of all fire dampers required by WNP-2 Technical Specifications were inspected and verified as installed.
- o An engineering study will be conducted to determine if a temporary 3 hour fire barrier should be installed in the duct from which ROA-FD-12 is missing as an interim corrective action.
- o ROA-FD-12 is being procured and will be installed and tested when received.
- o ROA-FD-12 will remain on the Fire Watch Tour until the new damper is operational or until the 3 hour fire barrier is installed.

Similar Events

None

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TEXT: If more space is required, use additional NRC Form 308A (1/77)

Safety Significance

If a design basis fire had occurred in the passageway outside the "B" RHR Heat Exchanger Room prior to the discovery that ROA-FD-12 was not installed, Division 1 safe shutdown wiring and equipment could have been damaged. If the fire had then traveled through the ventilation system and into the "B" RHR Heat Exchanger Room, damage to Division 2 safe shutdown wiring and equipment could have resulted. Sufficient damage to both Division 1 and Division 2 wiring and equipment could have resulted in the inability to safely shut down the plant. The probability of this sequence of events occurring is low, due to the fact that sufficient material to create the fire loading necessary for a design basis fire would have to be transported into the area via elevator or stairs. No fires occurred in the Reactor Building at WNP-2 during this period of time. This event did not affect the safety of the public or Plant personnel.

EIIS Information

Text Reference	EIIS Reference	
	System	Component
ROA-FD-12	VA	DMP
"B" RHR Heat Exchanger	BO	HX

Washington Public Power Supply System

3000 George Washington Way P.O. Box 968 Richland, Washington 99352-0968 (509)372-5000

Docket No. 50-367

November 5, 1985

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 85-057

Dear Sir:

Transmitted herewith is Licensee Event Report No. 85-057 for WNP-2 Plant. This report is submitted in response to the report requirements of 10 CFR 50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

Very truly yours,



C. M. Powers (M/D 927M)
WNP-2 Plant Manager

CMP/RDC:db

Enclosure:
Licensee Event Report No. 85-057

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (M/D 901A)
Ms. Dottie Sherman, ANI
INPO Records Center - Atlanta, GA
Mr. C. R. Bryant, BPA (M/D 399)

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