OPERATING DATA REPORT

DOCKET NO. 50-344
DATE
COMPLETED BY F. J. Ulmer
TELEPHONE 503-556-3713
Ext. 495

OPERATING STATUS

1. Unit Name: Trojan Nuclear Plant
2. Reporting Period: August 1985
3. Licensed Thermal Power (MWt): 3411
4. Nameplate Rating (Gross MWe): 1216
5. Design Electrical Rating (Net MWe): 1130
6. Maximum Dependable Capacity (Gross MWe): 1122
7. Maximum Dependable Capacity (Net MWe): 1080
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
   N/A
9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

Notes

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.	5,831.	78,887.
12. Number Of Hours Reactor Was Critical	735.24	4,039.8	46,785.5
13. Reactor Reserve Shutdown Hours	0.0	0.0	3,887.4
14. Hours Generator On-Line	733.8	3,958.9	46,294.4
15. Unit Reserve Shutdown Hours	0.0	0.0	3,249.
16. Gross Thermal Energy Generated (MWH)	2,491,756.	13,168,977.	146,138,226.
17. Gross Electrical Energy Generated (MWH)	804,782.	4,222,736.	47,778,516.
18. Net Electrical Energy Generated (MWH)	768,835.	4,013,062.	45,163,592.
19. Unit Service Factor	98.6	67.9	58.7
20. Unit Availability Factor	98.6	67.9	62.8
21. Unit Capacity Factor (Using MDC Net)	95.7	63.7	53.0
22. Unit Capacity Factor (Using DER Net)	91.4	60.9	50.7
23. Unit Forced Outage Rate	1.4	7.1	16.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
    N/A

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	N/A	N/A
INITIAL ELECTRICITY	N/A	N/A
COMMERCIAL OPERATION	N/A	N/A

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-344
UNIT: Trojan
DATE:
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TELEPHONE: (503) 556-3713
Ext. 495

MONTH August 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1060	17	1044
2	1051	18	1054
3	1055	19	1058
4	1060	20	1058
5	1060	21	1057
6	1059	22	1050
7	1062	23	1039
8	1058	24	1044
9	1062	25	1048
10	1056	26	527
11	1055	27	965
12	1052	28	1058
13	1044	29	1055
14	1048	30	1059
15	1050	31	1059
16	1039		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH August

DOCKET NO. 50-344
UNIT NAME Trojan
DATE
COMPLETED BY E. J. Ulmer
TELEPHONE (503) 556-3713 Ext 495

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
85-06	850826	F	10.2	G	3	85-10	EB	TRANSF	Reactor trip due to multiple 2-out-of-4 SSPS trip logics being met. Simultaneous to the performing of a periodic instrumentation and control test (PICT) on Reactor Protection Set IV, an instrument and control technician accidently shorted a heat trace circuit to an auxiliary feedwater flow instrumentation device which lowered the voltage on 120 vac preferred instrument bus Y13 sufficently to cause the bistables on Reactor Protection Set III to actuate thus causing a reactor trip. The shorted heat trace circuit was corrected. Trojan is reviewing the adequacy of the preferred instrument power supply inverters.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

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SUMMARY OF OPERATING EXPERIENCE

DOCKET NO: 50-344
DATE:
COMPLETED BY: F. J. Ulmer
TELEPHONE: 503-556-3713

OPERATION:

SEE ATTACHED

MAJOR SAFETY-RELATED MAINTENANCE:

1. Train A ESF equipment outage for scheduled routine maintenance.
2. Train B ESF equipment outage for scheduled routine maintenance.
3. Reset reactor low pressure trip, reactor safety injection low pressure trip setpoint, and pressurizer high level trip setpoint to 1890, 1855, and 88% respectively. Readjusted control pressure of RCS to 2255 psig indication.

MISCELLANEOUS MAINTENANCE:

1. Repaired containment condensate return valve (MO-1605)
2. Auxiliary feedwater pump recirculation line leak repaired.
3. Replaced valve manifold for FIS-3004A2 auxiliary feedwater flow.
4. Replaced speed switch on B Emergency Diesel Generator.

LICENSE CHANGES:

NONE

MISCELLANEOUS:

NONE

Operation:

The plant entered the month of August in Mode 1 at 100% power. Except for the turbine control valve testing on 2, 16, and 29 of August, no major power reductions occurred with the exception of the August 26 plant trip. Due to warm summer weather, slight decreases in turbine load were necessary due to high condenser back-pressure. The warm weather caused higher transformer temperatures on August 16 and main transformer spray cooling was initiated to maintain full power operation.

On August 21, the low pressure reactor trip setpoint was changed from 1865 to 1890 psig. The change resulted from inaccuracies in the instrument calibration from not including in the calculations the hydrostatic head of water above the instrument. On August 28, the low pressure safety injection setpoint (1835 psig) and the pressurizer high level trip setpoint (92%) were changed to 1855 psig and 88% respectively. The SI setpoint change was due to inaccurates in instrument calibration as stated above and the pressurizer high level trip setpoint was changed as a result of transmitter drift. The control setpoint for RCS pressure was increased by 20 psig.

At 1155 on August 26, the reactor automatically tripped from full load. The reactor trip was caused by an instrument and control technician accidently shorting a heat trace circuit on the flow instrument (FI-3043C2) while Reactor Protection Set IV was tripped for routine testing (PICT-11-1). The short caused a voltage dip on preferred 120 vac bus Y13 which supplies Reactor Protection Set III. The voltage dip caused several bistables to actuate in Protection Set III, thus completing 2-out-of-4 trip logic for multiple reactor trips in coincidence with Protection Set IV. The trip was immediately identified and corrective action taken to correct the heat trace circuit. The Trojan staff is presently re-evaluating the adequacy of preferred instrument power supplies.

At 2041 on August 26, the reactor was taken critical. At 2107, a turbine trip occurred during turbine roll testing from steam generator C water level swell which occurred when turbine control valve CV-2 came off its closed seat position. This trip caused a feedwater isolation signal, main feed pump trip, and auxiliary feedwater system automatic actuation. Steam generator levels were returned to normal and the main turbine was synchronized to the grid at 2207. The plant continued through the month at full power without incident.

Portland General Electric Company
Trojan Nuclear Plant
P.O. Box 439
Rainier, Oregon 97048
(503) 556-3713

September 5, 1985
WSO-563-85

Office of Resource Management
US Nuclear Regulatory Commission
Washington DC 20555

Gentlemen:

In accordance with the Trojan Nuclear Plant Technical Specifications reporting requirements, the monthly report is submitted for August, 1985.

Sincerely,

W. S. Orser
General Manager

WSO/GGB:pat

Attachment

c: Distribution
File 93.24b

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