BALTIMORE GAS AND ELECTRIC COMPANY

P.O. BOX 1475

BALTIMORE, MARYLAND 21203

NUCLEAR POWER DEPARTMENT
CALVERT CLIFFS NUCLEAR POWER PLANT
LUSBY, MARYLAND 20657

October 25, 1985

Dr. Thomas E. Murley
Regional Administrator
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

SUBJECT: **Calvert Cliffs Nuclear Power Plant Units 1 and 2**
Inoperability of Post-Accident Sampling System

Dear Dr. Murley:

This is an update to my letter to you, dated August 28, 1985, on the status of the Post-Accident Sampling System (PASS) and formal follow-up of a telephone discussion between Mr. T.T. Martin of your office and Mr. P. T. Crinigan on September 23, 1985 on the same subject.

My August 28, 1985 letter indicated that we expected to demonstrate satisfactory reliability of the PASS in-line analyzers and complete chemical matrix solution testing by September 6, 1985. The testing of the chemical analyses with the recommended test matrix, as well as the procedures and training on the chemical analyses for the PASS diluted grab sample, were satisfactorily completed by that date. However, the installed PASS has shown to be a maintenance intensive and unreliable system . In addition, while the analytical methods associated with the diluted grab sample have been demonstrated to be well within the recommended tolerances of NUREG 0737, the installed instrumentation inaccuracies often cause the analytical results to be outside the suggested tolerances up to a factor of three. Therefore, we believe neither the in-line analyzer system nor diluted grab sample and associated analyses can be considered a viable method for post-accident sampling and analyses.

Consequently, we are in the process of making modifications to the NSSS sample sink and to our laboratory sampling station to enchance our current method. These measures are expected to be completed within six weeks. In approximately one week we expect to submit these plans and description of analytical methods to NRR for their approval as our method of taking and analyzing a reactor coolant post accident sample. In the interim, we will continue to employ our back-up method, which utilizes the existing NSSS sample sink.

1/0

IE01

If you have any questions concerning the above, please give me a call.

LB Russell

L. B. Russsell
Plant Superintendent

LBR/djw

copies: Director, Office of Management
Information and Program Control
Mr. A. E. Lundvall, Jr.
Mr. W. J. Lippold