

Public Service
Electric and Gas
Company

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December 26, 1985

Director of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
7920 Norfolk Avenue
Bethesda, Maryland 20814

Attention: Ms. Elinor Adensam, Director
Project Directorate 3
Division of BWR Licensing

Dear Ms. Adensam:

RADIATION MONITORING SYSTEM COMPLETION STATUS
HOPE CREEK GENERATING STATION (HCGS)
DOCKET NO. 50-354

In response to your letter of November 21, 1985 concerning the status of the Hope Creek Radiation Monitoring System (RMS), Public Service Electric and Gas Company (PSE&G) hereby submits for NRC review Attachment I which details the current RMS completion status.

For those RMS components identified in Attachment I a safety evaluation will be performed to ensure that the proposed configuration of the components poses no undue risk to the health and safety of the public. Both the Station Operation Review Committee (SORC) and Offsite Safety Review Group (OSR) will review each safety evaluation. The results of these safety evaluations will be furnished to the NRC at least 30 days prior to the anticipated fuel load date.

Expedited review of this updated deferral request is needed to support PSE&G's efforts to load fuel at HCGS on February 15, 1986. Public Service Electric and Gas is ready to meet with the cognizant NRC personnel to review these matters should you require additional information.

Sincerely,

Add

C A McNeill Jr / JTB

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EB (LTAW)
PDB (L. HULMAN)
RICH (SRINIVASAN)
RSB (ACTING)
PDB (VASSALLO)
AD - G. LAINAS (ltr only)

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Director of Nuclear
Reactor Regulation

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Attachment

C D.H. Wagner
USNRC Licensing Project Manager

R.W. Borchardt
USNRC Senior Resident Inspector

J. Strosnider
USNRC Section Chief

H. Bicehouse
USNRC Radiation Specialist

ATTACHMENT I
RMS COMPLETION STATUS AT 5% POWER

The Following provides the Radiation Monitoring System (RMS) completion status at 5% power. Each item identified below is correlated to its respective roman numeral designator as identified in Attachment II of our September 10, 1985 submittal.

- I. Reactor Building Ventilation Exhaust (RBVSE) - RMS

The RBVSE radiation monitors will be functional locally prior to achieving 5% power.

- II. Turbine Building Exhaust (TBE), Turbine Building Compartment Exhaust (TBCE) and Gaseous Radwaste Area Exhaust (GRAE) - RMS

The TBE, TBCE, and GRAE radiation monitors will be functional locally prior to achieving 5% power.

- III. Radwaste Exhaust System (RES) and Radwaste Area Exhaust System (RAE) - RMS

The RES and RAE radiation monitors will be functional locally prior to achieving 5% power.

- IV. Technical Support Center Ventilation (TSCV) - RMS

The TSCV radiation monitors will be functional locally prior to achieving 5% power.

- V. Reactor Auxiliary Cooling System (RACS), Safety Auxiliary Cooling System (SACS), Heating Steam Condensate Decontamination System (HSCD), Turbine Building Circulating Water System (TBCW) - RMS

These radiation monitors will be functional locally prior to achieving 5% power.

- VI. Reactor Building Doors, Hatches and Airlocks -

The door alarm and monitoring system provides door status including hatches and airlocks (open/closed) via communication paths from the Local Radiation Processor (LRP) to the Central Radiation Processor (CRP). The door alarm and monitoring system portion of the CRP will not be available prior to exceeding 5% power (see item VIII).

However, PSE&G will monitor door status by utilizing the plant security system where applicable. The doors of concern are normally maintained in the closed position with the exception of the airlock. Administrative controls will be established prior to fuel load to ensure that appropriate plant personnel are informed of a change in door status from the central alarm station.

Since the details of this alternative method for monitoring door status affects the Hope Creek security plan, specifics will be provided for NRC review under separate cover.

VII. Drywell Leak Detection (DLD) - RMS

The radiation detection installed on the DLD radiation monitor at fuel load is only designed to operate at temperatures up to 120°F. A replacement radiation detector designed for 150°F will be available 150 days after fuel load.

VIII. Central Radiation Processor (CRP)

The CRP will be fully functional 180 days after fuel load. Prior or on this date, certain functions of the CRP will be available, however, these functions will not be available prior to 5% power. These CRP functions include the following:

1. The RM-11 computer, which acts as a data collector and provides the control room operators with measurements, alarms, and operational status for all radiation monitors, will be operational 120 days after fuel load.
2. The Micro Vax II computer which provides the Regulatory Guide 1.21 reports and the emergency dose assessment report will be operational 60 days after fuel load.
3. The balance of the CRP is comprised of the VAX 11/750 computer which provides input to the SPDS and CRIDS plant computer. The VAX 11/750 will be operational 180 days after fuel load.

IX. Emergency Response Facilities -

This issue is acceptable as documented in your letter of November 21, 1985.

X. Safety Parameter Display System (SPDS) -

All radiation monitoring system values and the suppression pool temperature monitoring will be functional as input to the SPDS 180 days after fuel load.

XI. Seismic Qualification Reports -

The seismic Qualification Reports for the Sample Flow Splitter skid for the Filtration Recirculation Ventilation System Vents and Suppression Pool Temperature Monitoring Input and Multiplex Circuit boards will be available prior to exceeding 5% power.

XII. Preoperational Testing -

Those RMS components identified as not being available at fuel load will be preoperationally tested in accordance with a Detailed Test Procedure and proven functional prior to service. The Detailed Test Procedure shall ensure that these RMS components meet the acceptance criteria presented in FSAR Sections 14.2.12.1.41 and 14.2.12.1.46.