

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Shoreham Nuclear Power Station Unit #1										DOCKET NUMBER (2) 0 5 0 0 0 3 2 2				PAGE (3) 1 OF 0 3								
TITLE (4) Update on Reactor Water Cleanup Isolations																						
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)									
0	9	0	3	8	5	8	5	0	3	6	0	1	1	1	0	6	8	5	0 5 0 0 0			
OPERATING MODE (9) 2			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following): (11)																			
POWER LEVEL (10) 0 1 0 1			20 402(b)				20 406(c)				<input checked="" type="checkbox"/> 60 73(a)(2)(iv)				73 71(b)							
			20 406(a)(1)(i)				60 36(c)(1)				60 73(a)(2)(iv)				73 71(c)							
			20 406(a)(1)(ii)				60 36(c)(2)				60 73(a)(2)(iv)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
			20 406(a)(1)(iii)				60 73(a)(2)(i)				60 73(a)(2)(viii)(A)											
			20 406(a)(1)(iv)				60 72(a)(2)(iv)				60 73(a)(2)(viii)(B)											
			20 406(a)(1)(v)				60 73(a)(2)(iii)				60 73(a)(2)(ix)											
LICENSEE CONTACT FOR THIS LER (12)																						
NAME Robert W. Grunseich, Operational Compliance Engineer										TELEPHONE NUMBER AREA CODE 5 1 6 9 2 9 - 8 3 0 0												
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																						
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS			
SUPPLEMENTAL REPORT EXPECTED (14)														EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR				
<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)														NO		1	2	3	1	8	5	
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single spaced, typewritten lines): (16)																						

Six (6) Reactor Water Cleanup (RWCU) system isolations occurred during the months of September and October 1985 while the plant was in various Operational Conditions. Three (3) RWCU inboard and outboard isolations occurred due to spurious high delta-flow isolation signals. These isolations may have resulted from a loose ground connection on the Square Root Extractor (G33-FK-Q13). The Square Root Extractor provides flow indication in the blowdown line to the Main Condenser. The loose ground connection was found while troubleshooting the RWCU system to determine the cause of the isolations. The ground was tightened. The cause of the three (3) inboard isolations is currently being investigated. Individual components in the RWCU inboard isolation circuitry are being examined to determine the cause of the isolations. After each event the system was returned to normal, Plant Management was notified and the NRC was notified per 10CFR50.72. For the events involving the inboard and outboard isolations, the cause of the isolations will be verified when the current source outage is complete and the plant reaches normal rated conditions. For the events involving the three inboard isolations, a supplemental report will be forthcoming once the cause has been determined and a solution has been identified. This supplemental report will provide an update on all six isolations.

IE 22
11

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1): Shoreham Nuclear Power Station Unit #1	DOCKET NUMBER (2): 0 5 0 0 0 3 2 2	LER NUMBER (6):			PAGE (3):	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 5	- 0 3 6	- 0 1	0 2	OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Six (6) Reactor Water Cleanup (RWCU) System isolations occurred during the months of September and October while the plant was in various Operational Conditions.

Three (3) RWCU system isolations occurred due to RWCU high delta-flow trips resulting in the closure of the inboard and outboard RWCU isolation valves (G33*MOV-033 and 034). The first isolation occurred on September 27, 1985 at 0432 while the plant was in Operational Condition 3 with the mode switch in shutdown and the RPV temperature at 235 degrees F. Operators had been controlling the RPV level utilizing the RWCU system, while adjusting the blowdown flow to the Main Condenser. The second isolation occurred on October 1, 1985 at 0357 while the plant was in Operational Condition 4 with the mode switch in shutdown and the RPV temperature at 149 degrees F. Technicians were performing recalibration of the RWCU delta-flow summer (which provides delta-flow indication of reactor water in versus reactor water out of the vessel). The third isolation occurred on October 7, 1985 at 2315 while the plant was in Operational Condition 2 with the mode switch in startup and the RPV temperature at 540 degrees F. Startup testing was being performed on the RWCU system prior to the event. All three isolations occurred due to spurious high delta-flow isolation signals. These isolations may have resulted from a loose ground connection on the Square Root Extractor (G33-FK-013). The Square Root Extractor provides flow indication in the blowdown line to the Main Condenser. After the second isolation, individual components in the RWCU flow sensing circuitry were recalibrated. However, after the third isolation additional trouble shooting of the RWCU system was performed to determine the cause of these isolations. At this time the loose ground on the Square Root Extractor was found and was tightened. After each event the system was returned to normal.

Three (3) other RWCU system isolations occurred which resulted in the closure of the inboard RWCU isolation valve (G33*MOV-033) only. The first two isolations occurred on September 3, 1985 at 2320 and on September 25, 1985 at 2215, while the plant was in Operational Condition 2 with the mode switch in startup and the RPV temperature at 240 degrees F and 538 degrees F respectively. Both isolations occurred while operators were adjusting the blowdown flow to the Main Condenser. The third isolation occurred on October 8, 1985 at 1253 while the plant was in Operational Condition 3 with the mode switch in shutdown and the RPV temperature at 260 degrees F. The isolation occurred just as the mode switch was placed into shutdown. Individual components in the RWCU inboard isolation circuitry are currently being examined to determine the cause of the isolations. After each event the system was returned to normal.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Shoreham Nuclear Power Station Unit #1	DOCKET NUMBER (2) 05000322	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		85	036	01	03	OF	03

TEXT (If more space is required, use additional NRC Form 366A p. (17))

There was no safety significance to the events. All other plant systems operated as designed. No ECCS systems were challenged or required for the events. The operators carried out all required actions. Plant Management was notified of the events and the NRC was notified per 10CFR50.72.

For the three events involving both the inboard and outboard isolations, the cause of the isolations will be verified once the current source outage is complete and the plant reaches normal rated conditions. For the three events involving the inboard isolations, a supplemental report will be forthcoming once the cause of the inboard isolations has been determined and a solution has been identified. This supplemental report will provide an update on all six isolations.



LONG ISLAND LIGHTING COMPANY

SHOREHAM NUCLEAR POWER STATION • P.O. BOX 628 • WADING RIVER, NEW YORK 11792

TEL. (516) 929-8300

November 6, 1985

PM-85-256

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

The enclosed revision to Licensee Event Report 85-036 is being submitted to provide an update on the Reactor Water Cleanup isolations which occurred in the months of September and October. Two additional events have been included in the revision because of their close relationship with the previous four events. This report provides revised information as to the cause of the events.

Sincerely yours,

William E. Steiger, Jr.
Plant Manager

WES/gr

Enclosure

cc: Dr. Thomas E. Murley, Regional Administrator
John Berry, Senior Resident Inspector
Institute of Nuclear Power Operations, Records Center
American Nuclear Insurers

SR. A21.0200

IE22
1/1