LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

OCTOBER, 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

IE24
1/1

I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy, and the primary construction contractor was Commonwealth Edision Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982, and commercial power operation was commenced on January 1, 1984.

This report was compiled by James P. Peters, telephone number (815)357-6761 extension 325.

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II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

October 1-31

Oct. 1, 0001 Hours	The unit entered October with the reactor critical at 77% power and coasting down with the generator on line at 830 MWe.
Oct. 15, 0700 Hours	Reactor Power coasting down at 72% (765 MWe).
Oct. 15, 0900 Hours	MDRFP tripped and reactor recirculation FCV runback to 36% power (400 MWe)
Oct. 15, 1800 Hours	Reactor power at 60% (620 MWe) holding for condenser in-leakage test.
Oct. 18, 0001 Hours	Began shutdown of reactor at a 50 MWe/Hr ramp down.
Oct. 18, 1125 Hours	Generator taken off-line.
Oct. 18, 1600 Hours	Manual scram
Oct. 31, 2400 hours	Reactor in cold shutdown for first refuel outage.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

Amendment 25-change to support delay of Unit #1 first refuel.

Amendment 26-change to:
1) Correction of typographical errors
2) Add Tech Spec. 3.0.4-Does not apply to Tech Spec 3.6.3
3) Revised RPIS Indication and surveillance requirements
4) Clarify Delay correction of liquid effluent
5) Clarify Emergency Make-up train heaters surviellance
6) RCIC Pump Room Isolation on differential temperature

Amendment 27-Add addition remote shutdown instrumentation to Technical Specification 3.3.7.4.

Amendment 28-Revise Setpoint and Setpoint Tolerance for Safety Relief Valves.

2. Facility or procedure changes requiring NRC approval.

There were no Facility or Procedure Changes Requiring NRC approval during this reporting period.

3. Tests and Experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during this reporting period.

4. Corrective maintenance of safety related equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L45720	SBGT Radiation Monitor Switch 0D18-S805	Control Switch was not wired per drawing.	Improper Annunciation	Rewired Per Sargent and Lundy Drawing.
L51279	APRM Power Supplies 1C51-K605	Defective Power Supply	Incomplete Surviellance LIS-NR-111	Replaced Power Supplies.
L51824	"1B" RHR Service Water Pump SWGR 202C, 1E12-C300B.	Dirty Interlock Mechanism, Linkage and Chain Drive.	Sleeve on Breaker will not Engage.	Cleaned Interlock Mechanism and Lubricated Friction Point of Sliding Bar.
L51880	OA"VE" Compressor 0VE04CA	3 Unloader Forks, 1 valve Plate and 1 set of suction Valves Were Bad or Missing	Compressor has low oil Pressure	Replaced 3 Unloader Forks, 1 Valve Plate, 1 Set of Suction Valves and 1 Partial Gasket Set.
L52226	RHR Minimium Flow Switch 1E12-N010CA	Defective Swith	Switch Would not Trip.	Replaced Switch, Pressure and Leak Tested, and Recalibrated.
L52386	HCU Charging Manifold Stop Isolation Valve 1C115835 F111	Damaged Packing within the Valve Body.	Valve Leakage Resulted in Difficulty of Charging Accum-mulator Header.	Replaced Damaged Packing.
L52590	"B" Chlorine VC Detector 0AE-VC091B	Bad Wick and Wet Carbon and Filter.	Detector Had Spurious Trips and would not reset.	Replace wick and dried wet Carbon and Filter.
L52741	Division I Post Loca Hydrogen 1AIT-CM009	Bad Hydrogen Cell and R27 Resistor	Hydrogen Channel would not meet span requirements.	Replaced Hydrogren cell and resistor R27.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L53000	"B" Narrow Range and Wide Range Level Indicator 1C34-N004B/ 1B21-N026B	Partially Filled Reference Leg.	Flashing indication in the Control Room Between 35 inches and 46 inches.	Isolated associated Switch and Backfilled Reference Legs.
L53024	Div-II, 125VDC Battery Charger	Bad Firing Module	Will Not Put Out Amperage Per Tech. Spec. 4.8.2.3.2.c.4 Failure of LES-DC-103.	Replaced Firing Module.
L53069	VC Ammonia Detector 0XY-VC-125A	Broken Chemcassette Tape.	Detector Upscale and Will Not Reset.	Replaced Chemcassette Tape.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, logged during the reporting period, October 1 through October 31, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence
85-63-00	10/8/85	Chlorine Detector "B" VC Trip.
85-64-00	10/14/85	Spurious trip of "A" ammonia detector on "B" VC Train.
85-65-00	9/29/85	VC/VE Chlorine Detector Actuation.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report

2. Average Daily Unit Power Level

3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LaSalle One
DATE November 9, 1985
COMPLETED BY James P. Peters
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: October 1985 GROSS HOURS IN REPORTING PERIOD: 745
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY): N/A

		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	424:00	5757:5	12039:00
6.	REACTOR RESERVE SHUTDOWN HOURS	0:0	476:2	1642:00
7.	HOURS GENERATOR ON LINE	419:25	5585:35	11642:25
8.	UNIT RESERVE SHUTDOWN HOURS	0:0	0:0	0:0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	988920	15390361	32213650
10.	GROSS ELEC. ENERGY GENERATED (MWH)	317976	5028751	10499394
11.	NET ELEC. ENERGY GENERATED (MWH)	300185	4827461	10022523
12.	REACTOR SERVICE FACTOR	56.9%	78.9%	74.8%
13.	REACTOR AVAILABILITY FACTOR	56.9%	85.4%	84.9%
14.	UNIT SERVICE FACTOR	56.3%	76.6%	72.3%
15.	UNIT AVAILABILITY FACTOR	56.3%	76.6%	72.3%
16.	UNIT CAPACITY FACTOR (USING MDC)	38.9%	63.9%	60%
17.	UNIT CAPACITY FACTOR(USING DESIGN MWe)	37.4%	61.4%	57.7%
18.	UNIT FORCED OUTAGE RATE	0%	17.4%	17.4%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH) The first refueling, maintenance, surviellance and modification outage began Octobr 18, 1985 and will last 26 weeks.
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: April 1986

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: November 9, 1985
COMPLETED BY: James P. Peters
TELEPHONE: (815) 357-6761

MONTH: OCTOBER 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	796	17.	597
2.	792	18.	127
3	789	19.	-15
4.	786	20.	-31
5.	783	21.	-17
6.	779	22.	-18
7.	775	23.	-15
8.	770	24.	-13
9.	765	25.	-13
10.	759	26.	-12
11.	754	27.	-12
12.	749	28.	-10
13.	741	29.	-13
14.	732	30.	-13
15.	573	31.	-11
16.	597		

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373
UNIT NAME LaSalle One
DATE NOVEMBER 9, 1985
COMPLETED BY James P. Peters
TELEPHONE (815)357-6761

REPORT MONTH OCTOBER 1985

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
17	851015	S	0.0	F	5	Power Reduction For Condenser IN Leakage Test.
18	851018	S	325.58	C	2	Manual Reduction in Power For Refueling Outage. Manual Scram For SDV (Scram Discharge Volume) Testing.

DOCUMENT 0044r/0005r

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

DATE	VALVES ACTUATED	NO & TYPE ACTUATION	PLANT CONDITION	DESCRIPTION OF EVENT

There were no Safety Relief Valves Operated for Unit One during this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
1-784-85	1E12-C300B	Inspect circuit breaker
1-788-85	LPCS Pump	Flush Motor Cooler
1-799-85	1B DG Cooling Water Pump	Polarization Indexing
1-800-85	1B DG	Calibration of crankcase pressure switch.
1-826-85	"A" RHR Shutdown Cooling and LPCI	LES-RH-100
1-832-85	B/C RHR Shutdown Cooling and LPCI	LES-RH-101
1-853-85	1A DG	Refueling Outage
1-854-85	1A DG, 1DG01P	Inspect impeller
1-855-85	1A DG, 1DG01P	Replace bearings
1-856-85	1A DG	LES-DG-101
1-857-85	1A DG	Lube Cooling Pump
1-863-85	1E12-F092A/41B/41C	Resplice Cable.
1-865-85	"B" RHR Return to Fuel Pool	Install Spool Piece
1-866-85	"B" RHR Suction from Fuel Pool	Install Spool Piece
1-868-85	1E12-F005	Replace cable.
1-872-85	1A DG	LIS & LIP Procedures
1-878-85	"C" RHR Service Water Pump	Lube Coupling
1-887-85	1E12-C300D	Lube Coupling
1-908-85	1E12-F042B	LOP-RH-15
1-914-85	1A DG A.C. Oil Pump	Hot alignment on A.C. Oil Pump

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
1-919-85	B/C LPCI Injection Stops	LTS-800-2
1-933-85	1B DG	Inspection
1-934-85	1B DG Fuel Oil Transfer System	Inspection
1-935-85	1B D/G	LES-DG-101
1-936-85	1B D/G	LIS-DG-01
1-938-85	HPCS Pump	LES-HP-102
1-960-85	1E12-F009	Limitorque Inspection
1-968-85	1E12-F052B	Inspect Wiring
1-969-85	1E12-F006B	Inspect Wiring
1-971-85	"1B" DG A.C. Lube Oil Pump	Repair Leaks

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculation Manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no significant changes to the radioactive waste treatment system during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

OCTOBER 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

DOCUMENT ID 0036r/0005r

I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy, and the primary construction contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984, and commercial power operation was commenced on June 19, 1984.

This report was compiled by James P. Peters, telephone number (815)357-6761 extension 325.

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II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

October 1-31

Oct. 1, 0001 Hours	The Unit entered October with the reactor critical at 100% power and generator on line at 1112 MWe.
Oct. 1, 1008 Hours.	Reactor power reduced to 95% (1050 MWe) to exchange condensate pumps.
Oct. 1, 1500 Hours	Reactor Power Restore to 100% (1101 MWe).
Oct. 4, 0245 Hours	Reactor Power Reduced to 95% (1050 MWe) to exercise control rods.
Oct. 4, 0700 Hours	Reactor Power Restored to 100% (1104 Mwe)
Oct. 6, 0000 Hours	Reactor Power Reduced to 89% (984 MWe) per MSIV/TSV Testing and held by load dispatcher.
Oct. 7, 0200 Hours	Reactor Power Restored to 100% (1112 MWe)
Oct. 8, 1300 Hours	Reactor Power Reduced to 97% (1071 MWe) for control rod manipulation.
Oct. 8, 2300 Hours	Reactor Power Restored to 100% (1116 MWe)
Oct. 11, 0000 Hours	Reactor Power Reduced to 94% (1038 MWe) for control rod drive exercising.
Oct. 11, 2300 Hours	Reactor Power Restored to 100% (1100 MWe)
Oct. 17, 0800 Hours	Reactor Power Reduced to 96% (1065 MWe) for control rod manipulation.
Oct. 17, 1500 Hours.	Reactor Power Restored to 100% (1100 MWe)
Oct. 18, 0000 Hours	Reactor Power Reduced to 95% (1054 MWe) for control rod manipulation.
Oct. 18, 0700 Hours	Reactor Power Restored to 100% (1100 MWe)
Oct. 21, 1251 Hours	Reactor Scram on Spurious Main Steam Line Isolation
Oct. 31, 2400 Hours	Reactor in Cold Shutdown for Short Outage.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specifications.

Amendment 13-To support delay for first refuel outage on Unit #1.

Amendment 14-change to:
1) Correction of typographical errors
2) Add Tech Spec. 3.0.4-Does not apply to Tech Spec. 3.6.3
3) Revised RPIS Indication and Surviellance Requirements
4) Clarify Delay correction of liquid effluent
5) Clarify Emergency Make-up train heaters Surviellance
6) Add RCIC Pump Room Isolation on Differential Temperature

Amendment 15-Revise Setpoint and Setpoint Tolerance for Safety Relief Valves.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure change requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit Two during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L51444	HPCS Water Leg Pump 2E22-C003	Adjusting Impeller Per Vendor Manual	No Significant Effect, Check Volume Was Filled Every Hour.	Adjusted Impeller
L52278	APRM-ION Changer, Power Supply.	Bad Diodes in the Power Supply.	Excessive Ripple and Low Output of Power Supply.	Installed new Diodes.
L52300	Scram Discharge Volume Level Switch 2C11-N013H.	Microswitch Would Not Completely Plunge Upon Actuation.	Alarm Does Not Actuate.	Replaced Level Switch.
L52350	HPCS Lo Lo Level Initiation Switch 2B21-N031D.	Broken and Punctured Wire at Switch.	Switch Exhibits Large Positive Ground While in Circuit.	Retaped and Repaired Wire.
L52436	"B" VE Return Fan C/B 0VE-02CB	Thermal Overload Reset Button Would Not Reset.	Fan Will Not Reset.	Repaired Thermal Over-Load Button.
L52438	"B" VE Refrigerant Oil Oil Temperature Indicator 0TS-VE141	Bad Temperature Switch	Temperature Switch Inoperative.	Replaced With New Temperature Switch.
L52443	HPCS Pump 2E22-C002 Breaker MCC-243-1 Compt. 1D.	Bad Breaker	When Starting The Pump, The Breaker Would Occasionally Trip.	Replace Breaker.
L52564	2B DG Cooling Water Pump 2E22-C002 Breaker 243-1-1D.	Change Per Modification 1-2-85-062	More Reliable Breaker Operation.	Change Magnetic Setting From 6 to 6.5.
L52889	RBM "A"	Bad 5 Volt Power Supply	A.C. Ripple Greater Than 150 mV.	Replace 5 Volt Power Supply.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L53026	Scram Discharge South Volume Level Hi Alarm 2C11-N013H.	Misaligned Actuating Arm.	Failure of LIS-RD-401, Hi Level Alarm Did Not Actuate.	Adjusted Actuating Arm.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, logged during the reporting period, October 1 through October 31, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence
85-041-00	9/30/85	HPCS 2B21-N031D Low Level Initiation Switch Malfunction.
85-042-00	10/04/85	HPCS Diesel Cooling Water Pump Breaker Defective.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report

2. Average Daily Unit Power Level

3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LaSalle Two
DATE November 9, 1985
COMPLETED BY James P. Peters
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: October, 1985 GROSS HOURS IN REPORTING PERIOD: 745
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net):1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY): N/A

		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	492:51	3558.8	5170.6
6.	REACTOR RESERVE SHUTDOWN HOURS	252.49	316.6	441.9
7.	HOURS GENERATOR ON LINE	492:51	3507.81	5045.21
8.	UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	1610640	10567126	15074718
10.	GROSS ELEC. ENERGY GENERATED (MWH)	536160	3485449	4970435
11.	NET ELEC. ENERGY GENERATED (MWH)	515066	3311157	4703474
12.	REACTOR SERVICE FACTOR	66.1%	48.8%	56.8%
13.	REACTOR AVAILABILITY FACTOR	100%	53.1%	61.7%
14.	UNIT SERVICE FACTOR	66.1%	38.6%	55.5%
15.	UNIT AVAILABILITY FACTOR	66.1%	38.6%	55.5%
16.	UNIT CAPACITY FACTOR (USING MDC)	66.7%	43.8%	49.9%
17.	UNIT CAPACITY FACTOR(USING DESIGN MWe)	64.1%	42.1%	47.9%
18.	UNIT FORCED OUTAGE RATE	8.4%	8.6%	11.4%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): On October 21, 1985 an outage for unfinished EQ (Environmental qualification) Modifications and Limitorque Valve Inspections Began.
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP November 23, 1985

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
UNIT: LASALLE TWO
DATE: November 9, 1985
COMPLETED BY: James P. Peters
TELEPHONE: (815) 357-6761
MONTH: August 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	1060	17.	1047
2.	1060	18.	1054
3	1060	19.	1057
4.	1058	20.	1054
5.	1060	21.	554
6.	988	22.	-17
7.	1065	23.	-29
8.	1057	24.	-13
9.	1066	25.	-12
10.	1060	26.	-11
11.	1045	27.	-12
12.	1057	28.	-11
13.	1051	29.	-11
14.	1046	30.	-12
15.	1045	31.	-11
16.	1043		

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
UNIT NAME LaSalle Two
DATE November 9, 1985
COMPLETED BY James P. Peters
TELEPHONE (815)357-6761

REPORT MONTH OCTOBER 1985

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
11	851021	F	252.15	H	3	Reactor Scram on Spurious Steam-Line Isolation During an Instrument Maintenance Surviellance.

DOCUMENT 0044r/0005r

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

DATE	VALVES ACTUATED	NO & TYPE ACTUATIONS	PLANT CONDITION	DESCRIPTION OF EVENT
10-21-85	2B21-F013U	17 Automatic	Hot Shutdown	Opened to Relieve Pressure Following Main Steam Line Isolation and Reactor Scram.
10-21-85	2B21-F013S	1 Automatic	Hot Shutdown	" "
10-21-85	2B21-F013D	1 Automatic	Hot Shutdown	" "
10-21-85	2B21-F013A	1 Manual	Hot Shutdown	" "
10-21-85	2B21-F013B	1 Manual	Hot Shutdown	" "
10-21-85	2B21-F013C	1 Manual	Hot Shutdown	" "
10-21-85	2B21-F013E	1 Manual	Hot Shutdown	" "

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
2-1221-85	2A DG	Meggar DG Cooling Pump
2-1222-85	2E12-F024B/47B/48B	EQ (Environmental Qualification) Motor Replacements.
2-1223-85	HPCS and Div III	Investigate 2B DG Cooling Water Pump Breaker.
2-1224-85	HPCS and Div-III	Trouble Shout 2B DG Cooling Water Pump Breaker.
2-1226-85	2E12-F006A	Replace Grease.
2-1227-85	2E12-F006A	Electrical Maintenance Assist to Mechanical Maintenace.
2-1228-85	2E12-F064A	Electrical Maintenance Assist to Mechanical Maintenance.
2-1229-85	2E12-F064A	Replace Grease.
2-1232-85	2B DG	Calibrate Fuel Pump Alarm.
2-1240-85	2E12-F048A	Replace Motor and Inspect Wiring.
2-1244-85	2E22-C003	Adjust Pump Impeller.
2-1264-85	2E12-F008	Inspect Limitorque Wiring.
2-1302-85	2E12-F009	Inspect Limitorque Wiring.
2-1315-85	2E22-F004	Inspect Limitorque Wiring.
2-1328-85	2E12-F016B	Inspect Limitorque Wiring.
2-1329-85	2E12-F016A	Inspect Limitorque Wiring.

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculation manual during this reporting period.

4. Radioactive Waste Treatment Systems.
There were no changes to the radioactive waste treatment system during this reporting period.

Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

November 7, 1985

Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for the period October 1 through October 31, 1985.

Very truly yours,

G. J. Diederich
Station Manager
LaSalle County Station

GJD/RJR/crh

Enclosure

xc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
D. P. Galle, CECo
D. L. Farrar, CECo
INPO Records Center
Ron A. Johnson, PIP Coordinator SNED
J. E. Ellis, GE Resident
J. M. Nowicki, Asst. Comptroller
H. E. Bliss, Nuclear Fuel Services Manager
C. F. Dillon, Senior Financial Coordinator, LaSalle

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