

NRC FORM 366
(12-81)U.S. NUCLEAR REGULATORY COMMISSION
LICENSEE EVENT REPORTAPPROVED BY OMB
3150-0011
EXPIRES 4-30-82CONTROL BLOCK: ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01	M	D	C	C	N	2	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4		5																
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34																
LICENSEE CODE														LICENSE NUMBER										LICENSE TYPE										CAT 58									

CON'T

01	L	0	5	0	0	0	3	1	8	7	0	2	0	7	8	3	8	1	1	0	8	8	5	9							
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34				
REPORT SOURCE		DOCKET NUMBER										EVENT DATE										REPORT DATE									

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | At 1045, during surveillance testing, ESFAS A logic sequencer failed

03 | rendering #12 Diesel Generator out of service to Unit 2 (T.S. 3.8.1.1).

04 | The remaining A.C. electrical power sources were verified to be operable

05 | at 1121. The sequencer was replaced restoring #12 Diesel Generator to

06 | service to Unit 2 at 1350.

07 | Similar events: 50-317/82-85

09	I	B	11	E	12	G	13	I	N	S	T	R	U	14	X	15	Z	16											
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25											
SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE								COMP. SUBCODE		VALVE SUBCODE													
LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.		ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER	
17		21		24		27		30		32		33		34		35		36		37		40		42		43		44	
18		19		20		21		22		23		24		25		26		27		28		29		30		31		32	
C		Z		Z		Z		0		0		0		0		N		Y		N		V		1		3		2	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | Troubleshooting did not reveal the cause of the sequential blocking module

11 | failure (Vitro Labs N-04341.26023). The module was replaced with a spare.

12 | The failed module was sent to the vendor for repair and testing. Vendor

13 | failure analysis revealed no further corrective action required.

14 |

15	E	28	1	0	0	29	NA	30	B	31	Surveillance Test	32
7	8	9	10	11	12	13	14	15	16	17	18	19
FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION				
28		29		30		31		32				
16		17		18		19		20				
Z		Z		NA		NA		NA				
33		34		35		36		37				
33		34		35		36		37				
PERSONNEL EXPOSURE NUMBER		TYPE		DESCRIPTION		39		40				
17		18		19		20		21				
0		0		0		Z		NA				
37		38		39		40		41				
PERSONNEL INJURIES NUMBER		DESCRIPTION		41		42		43				
18		19		20		21		22				
0		0		0		NA		NA				
40		41		42		43		44				
LOSS OF OR DAMAGE TO FACILITY TYPE		DESCRIPTION		43		44		45				
19		20		21		22		23				
Z		NA		NA		NA		NA				
42		43		44		45		46				
PUBLICATION ISSUED		DESCRIPTION		45		46		47				
20		21		22		23		24				
N		NA		NA		NA		NA				
44		45		46		47		48				

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PDR ADOCK 05000318
S PDRIE22
VI

NAME OF PREPARER R. B. Sydnor

PHONE: 301-260-4014

NRC USE ONLY

BALTIMORE GAS AND ELECTRIC COMPANY

P.O. BOX 1475

BALTIMORE, MARYLAND 21203

NUCLEAR POWER DEPARTMENT
CALVERT CLIFFS NUCLEAR POWER PLANT
LUSBY, MARYLAND 20657

November 8, 1985

U. S. Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Docket No. 50-318
License No. DPR 69

Dear Sirs:

The attached LER 83-011/3X, Revision 1, is being sent to you as required by 10 CFR 50.73.

Should you have any questions regarding this report, we would be pleased to discuss them with you.

Very truly yours,

L B Russell

L. B. Russell
Plant Superintendent

^{MS}
LBR:RBS:fld

cc: Dr. Thomas E. Murley
Director, Office of Management Information
and Program Control

Messrs: A. E. Lundvall
J. A. Tiernan
W. J. Lippold

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