

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3										DOCKET NUMBER (2) 0 5 0 0 0 3 6 2				PAGE (3) 1 OF 01					
TITLE (4) FUEL HANDLING ISOLATION SYSTEM (FHIS) ACTUATIONS																			
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)						
10	08	85	85	031	001	10	06	85					0 5 0 0 0						
OPERATING MODE (9) 6		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																	
POWER LEVEL (10) 0 0 0		20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)					
		20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)					
		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)					
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)									
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)									
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)									
LICENSEE CONTACT FOR THIS LER (12)																			
NAME H. E. MORGAN, STATION MANAGER										TELEPHONE NUMBER 7 1 4 3 6 8 - 6 2 4 1									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																			
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
SUPPLEMENTAL REPORT EXPECTED (14)														EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR	
YES (If yes, complete EXPECTED SUBMISSION DATE)														<input checked="" type="checkbox"/> NO					

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 10/8/85 at 1110, with Unit 3 in Mode 6 during refueling operations, Fuel Handling Isolation System (FHIS) (EIIS System Code VG) was actuated by a signal from the Fuel Handling Building (FHB) Vent Airborne Process Radiation Monitor 3RI-7823 (EIIS Component Code RIT). On 10/10/85 at 1945 and 2005 and on 10/28/85 at 1102, 1117 and 1142, the FHIS was again actuated by signals from radiation monitors 3RI-7823 and 3RI-7822, respectively. All FHIS system components actuated as required.

The cause of the actuations was the increase of low level activity in the Fuel Handling Building resulting from the refueling operations, fuel movement and fuel reconstitution activities. The increased activity results from the amount of leaking fuel being removed. As reported in LERs 85-023 and 85-028 (Docket No. 50-362), actuation of the FHIS monitors are expected to decrease after completion of fuel reconstitution and refueling operations. In addition, the feasibility of revising the Technical Specifications to increase the FHIS monitor actuation setpoints is being evaluated to prevent non-accident low level radiation actuations.

The health and safety of plant personnel or the public was not affected by this event. Since all systems actuated as required, there are no reasonable or credible circumstances under which this event would have been more severe.

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LICENSEE EVENT REPORT (LER)

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TITLE (4) FUEL HANDLING ISOLATION SYSTEM (FHIS) ACTUATIONS															

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)			
10	08	85	85	0311	00							0 5 0 0 0			
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			20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
			20.405(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(viii)(A)						
			20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)						
			20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)						

LICENSEE CONTACT FOR THIS LER (12)										TELEPHONE NUMBER					
NAME H. E. MORGAN, STATION MANAGER										AREA CODE 714 368-6241					

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO				

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On 10/8/85 at 1110, with Unit 3 in Mode 6 during refueling operations, Fuel Handling Isolation System (FHIS) (EIIS System Code VG) was actuated by a signal from the Fuel Handling Building (FHB) Vent Airborne Process Radiation Monitor 3RI-7823 (EIIS Component Code RIT). On 10/10/85 at 1945 and 2005 and on 10/28/85 at 1102, 1117 and 1142, the FHIS was again actuated by signals from radiation monitors 3RI-7823 and 3RI-7822, respectively. All FHIS system components actuated as required.

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The health and safety of plant personnel or the public was not affected by this event. Since all systems actuated as required, there are no reasonable or credible circumstances under which this event would have been more severe.



Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION

P. O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

H. E. MORGAN
STATION MANAGER

TELEPHONE
(714) 368-6241

November 6, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-362
30-Day Report
Licensee Event Report No. 85-031
San Onofre Nuclear Generating Station, Unit 3

Pursuant to 10 CFR 50.73(a)(2)(iv), this submittal provides the required 30-day written Licensee Event Report (LER) for occurrences involving six actuations of the Fuel Handling Isolation System (FHIS). Neither the health and safety of plant personnel nor the health and safety of the public was affected by this event.

If you require any additional information, please so advise.

Sincerely,

H. E. Morgan

Enclosure: LER No. 85-031

cc: F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)

J. B. Martin (Regional Administrator, USNRC Region V)

Institute of Nuclear Power Operations (INPO)

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