

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)

River Bend Station, Unit 1

DOCKET NUMBER (2)

0 5 0 0 0 4 5 8 1 OF 0 3

TITLE (4)

Loss of Feedwater and Reactor Scram

EVENT DATE (6)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)				
1	1	2	1	8	5	8	5	0	4	1	0	5	0	0	0
1	1	2	1	8	5	8	5	0	4	1	0	5	0	0	0

OPERATING MODE (9) 2

POWER LEVEL (10) 0 0 3

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)

20.402(b)	20.405(a)	60.73(a)(2)(iv)	73.71(b)
20.405(a)(1)(i)	60.73(a)(1)	60.73(a)(2)(v)	73.71(a)
20.405(a)(1)(ii)	60.73(a)(2)	60.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 308A)
20.405(a)(1)(iii)	60.73(a)(2)(i)	60.73(a)(2)(vii)(A)	
20.405(a)(1)(iv)	60.73(a)(2)(ii)	60.73(a)(2)(vii)(B)	
20.405(a)(1)(v)	60.73(a)(2)(iii)	60.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME

Dan Williamson, Operations Supervisor

TELEPHONE NUMBER

AREA CODE

5 0 4 6 3 Ext. 2558

5 0 4 6 3 5 1 6 1 0 9 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)

X NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On 11/21/85 at 0122 with the unit in operational condition 2 (startup), a reactor scram occurred from 2.7% power on low reactor water level. The low level resulted from a trip of the reactor feedwater pump and inability to place a standby feedpump in service prior to the level decrease to the low level scram trip setpoint. Level was manually restored by Reactor Core Isolation Cooling at a minimum level of +2 inches.

There was no impact on the health and safety of the public.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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					YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
					— 0	4 1	— 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A (1/77))

On 11/21/85 at 0025 with the unit in operational condition 2 (startup), Reactor Feedwater Pump (RFP) B tripped for unknown causes. The operator restarted the pump and restored reactor water level to normal. In order to investigate the cause, RFP C was started with its discharge Motor Operated Valve (MOV 26C) left shut in the event RFP B tripped again. At 0117 RFP B tripped when its auxiliary oil pump was secured. The operator mistakenly believed that a normal oil supply was available. An attempt to open the discharge MOV for RFP C failed with the breaker tripping on overload. RFP B and its auxiliary oil pump was restarted, but an attempt to open its discharge valve also failed. At 0123 the reactor scrambled when level decreased to the low level scram setpoint. Reactor Core Isolation Cooling (RCIC) (IEEE:BN) was manually started to restore reactor level. The lowest level indicated on narrow range instrumentation was +2 inches (164 inches above TAF). At 0127 the RFP B discharge valve was opened and RCIC secured.

The cause of the initial RFP B trip is believed to have resulted from a main oil pump trip on overload due to low oil temperature. Investigation into the inability to open RFP B and C discharge valves determined that the cause was due to high differential pressure across the discharge valves. This would result when the feedwater to condenser recirculation valve (FV104) is open as it was in this case to aid in reactor level control.

In an effort to prevent recurrence, system operating procedure SOP-009 "Reactor Plant Feedwater" now has a caution inserted concerning RFP discharge valve operation during low power operations which could result in unusually high differential pressure. The operations staff

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River Bend Station, Unit 1	0500045885	04	1	00	03	OF 03

TEXT (if more space is required, use additional NRC Form 305A's) (17)

has also been issued a memorandum reminding personnel of the operation of the RFP oil system. The system will be addressed in the next phase of licensed operator requalification training beginning 01/03/86.

The health and safety of the public was not affected during this event.



**GULF STATES UTILITIES COMPANY**

RIVER BEND STATION      POST OFFICE BOX 220      ST. FRANCISVILLE, LOUISIANA 70775  
AREA CODE 504      635-6094      346-8651

December 21, 1985  
RBG- 22865  
File Nos. G9.5, G9.25.1.3

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Dear Sir:

River Bend Station - Unit 1  
Docket No. 50-458

Please find enclosed Licensee Event Report No. 85-041 for River Bend Station - Unit 1. This report is submitted pursuant to 10CFR50.73.

Sincerely,

J. E. Booker  
Manager-Engineering,  
Nuclear Fuels & Licensing  
River Bend Nuclear Group

*TFP* *BEH*  
JEB/TFP/PDG/BEH/ebm

cc: U.S. Nuclear Regulatory Commission  
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Arlington, TX 76011

INPO Records Center  
1100 Circle 75 Parkway  
Atlanta, GA 30339-3064

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