

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Waterford Steam Electric Station Unit 3										DOCKET NUMBER (2) 0 5 0 0 0 3 8 2					PAGE (3) 1 OF 0 3										
TITLE (4) Wet Cooling Tower Basin Level Instrument Discrepancy																									
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)												
									N/A				0 5 0 0 0												
1	1	2	6	8	5	8	5	0	4	9	0	0	1	2	2	3	8	5	N/A				0 5 0 0 0		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																							
1		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)											
POWER LEVEL (10)		20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)											
0 9 8		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)											
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)															
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)															
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)															
LICENSEE CONTACT FOR THIS LER (12)																									
NAME										TELEPHONE NUMBER															
T. Smith, Maintenance Superintendent										AREA CODE		5 0 4 4 6 4 - 3 1 1 3 8													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs															
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR											
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO															

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

At 1438 hours on November 26, 1985 Waterford Steam Electric Station Unit 3 was at 98% reactor power when plant personnel discovered a calculation error in the Instrument and Control Work Packages for the Auxiliary Component Cooling Water (EIIIS System Code BS) Wet Cooling Tower Basin Level (Component Identifier CTW) instrumentation. As a result, the indicated basin level, as measured in the Control Room, was below the Technical Specification level.

Upon discovery plant operators restored the basin water level above the Technical Specification level. The discrepancy has since been corrected.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Waterford 3 Steam Electric Station	DOCKET NUMBER (2) 0 5 0 0 0 3 8 2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	— 0 4 9	— 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

NARRATIVE

At 1438 hours on November 26, 1985 Waterford Steam Electric Station Unit 3 was at 98% reactor power when plant personnel discovered a calculation error in the Instrument and Control Work Packages for the Auxiliary Component Cooling Water (EIIIS System Code BS) Wet Cooling Tower Basin Level (Component Identifier CTW) instrumentation. As a result, the indicated basin level, as measured in the Control Room, was below the actual level. Although the water volume in the basin was greater than the volume for the most limiting event described in the Final Safety Analysis Report, the water level was below that specified in Technical Specification 3.7.4 (97%). Therefore, when the indicated level was 97% (from the Control Room) the actual level was approximately 95%.

Upon discovery plant operators restored the basin water level above the Technical Specification level.

SAFETY CONSEQUENCES AND IMPLICATIONS

Although the basin water level for both Wet Cooling Towers was below the Technical Specification level for a time longer than allowed by Technical Specifications, the volume of water in the basin was greater than the volume for the most limiting event described in the Final Safety Analysis Report. Since the water level described in the Technical Specifications is based on the water volume described in the Final Safety Analysis plus a maximum instrument error, it is felt that the above event in no way affects plant safety.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
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Waterford 3 Steam Electric Station	0 5 0 0 0 3 8 2 8 5 —	0	4	9	—	0	0 3 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

CORRECTIVE ACTIONS

A review of the work package, along with the instrumentation configuration, revealed that the level instruments in question were actually located at a higher elevation than assumed in the loop work package. Plant personnel have since corrected this discrepancy and issued the necessary paperwork to revise the affected documentation.

SIMILAR EVENTS

None

PLANT CONTACT

T. Smith, Maintenance Superintendent, 504/464-3138



LOUISIANA
POWER & LIGHT

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December 23, 1985

W3P85-3801
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QA

Director, Office of Nuclear Reactor Regulation
ATTENTION: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Waterford 3 SES
Docket No. 50-382
License No. NPF-38
Reporting of Licensee Event Report

Dear Sirs:

Attached is Licensee Event Report Number LER-85-049-00 for Waterford 3.
This Licensee Event Report is submitted per 10CFR50.73(a)(2)(i).

Very truly yours,

K.W. Cook
Nuclear Support & Licensing Manager

KWC:GEW:sms

Attachment

cc: R.D. Martin, G.W. Knighton, J.H. Wilson, NRC Resident Inspectors
Office, INPO Records Center (J.T. Wheelock), B.W. Churchill,
W.M. Stevenson

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