

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) EDWIN I. HATCH, UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 1	PAGE (3) 1 OF 0 2
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TITLE (4)
PRIMARY CONTAINMENT ISOLATION VALVE UPLANNED ISOLATION

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
1	1	17	8	5	0 3 9	0	0	12	1	7	8 5
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											0 5 0 0 0

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)									
POWER LEVEL (10) 0 9 9	20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)						
	20.406(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)						
	20.406(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	20.406(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)							
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)							
	20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)							

LICENSEE CONTACT FOR THIS LER (12)

NAME Steven B. Tipps, Superintendent of Regulatory Compliance	TELEPHONE NUMBER 9 1 1 2 3 6 7 1 1 7 8 5 1
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	X	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

This report describes the actuation of an Engineered Safety Feature and is submitted pursuant to 10CFR 50.73(a)(2)(iv).

On 11/17/85 at approximately 1423 CST, the unit was in steady-state operation at 2421 Mwt (approximately 99% rated power) and non-licensed personnel were properly returning Reactor Water Cleanup (RWC) non-regenerative heat exchanger temperature switch (1G31-N008) to service when a false high temperature trip signal was received from that sensor. This resulted in the isolation of the RWC outboard primary containment isolation valve (1G31-F004) and subsequent trip of the pump in that system.

An evaluation of system design, procedural controls, and personnel actions associated with this incident has revealed no physical cause for the false temperature trip. This event was apparently caused by a false electrical signal of unknown origin.

There is no corrective action to prevent false temperature trips of this nature.

The RWC pump and outboard primary containment isolation valve responded as required. Therefore, this event did not adversely affect plant safety or the health and safety of the public.

The spurious spike described in LER 50-321/1985-33 is similar to this event.

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PDR ADOCK 05000321
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
EDWIN I. HATCH, UNIT 1	05000321	85	039	00	02	OF 02

TEXT (If more space is required, use additional NRC Form 368A's) (17)

On 11/17/85 at approximately 1423 CST, the unit was in steady-state operation at 2421 MWt (approximately 99% rated power) and non-licensed personnel were properly returning Reactor Water Cleanup (RWC) non-regenerative heat exchanger temperature switch (1G31-N008) to service per the "FENWALL TYPE 56100-1/551 TEMPERATURE SWITCH" calibration procedure (57CP-CAL-065-1) when a false temperature trip was received from 1G31-N008. This resulted in the isolation of the RWC outboard primary containment isolation valve (1G31-F004) and subsequent trip of the pump in that system.

An investigation was initiated in an attempt to ascertain the root cause(s) for this incident. An engineering evaluation of the adequacy of control provided by 57CP-CAL-065-1 was conducted, as well as a review of the control logic for the RWC system. No deficiencies were identified in either the procedure or the system design which could have caused the false trip signal during calibration of 1G31-N008. Interviews with the plant personnel who performed the subject calibration failed to reveal any deviation from the referenced procedure or other action which could have resulted in the generation of the false high temperature signal. However, personnel closing the electrical links which returned 1G31-N008 to service in accordance with 57CP-CAL-065-1 stated that a small electrical arc was observed during the process. This arc occurred just as the final link was closed before the link screw was tightened.

As a result of this investigation, the false trip signal generated by 1G31-N008 has been attributed to a spurious electrical signal of unknown origin. The generation of a false electrical signal has been known to occur in other systems (for example see LER 50-321/1985-33). Due to the lack of known similar previous incidents involving the RWC logic, it is not believed that this specific type of event will be a recurring problem at Plant Hatch. Therefore, no corrective actions to prevent future occurrences are planned.

The RWC pump and outboard primary containment isolation valve responded as required. Therefore, this event did not adversely affect plant safety or the health and safety of the public.

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L. T. Gucwa
Manager Nuclear Safety and
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SL-101
0175C

December 17, 1985

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-321/1985-039. This report meets the reporting requirements of 10CFR 50.73(a)(2)(iv).

Very truly yours,

L. T. Gucwa

CBS/lc

Attachment

c: Mr. J. T. Beckham, Jr.
Mr. H. C. Nix, Jr.
NRC-Region II
GO-NORMS

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