

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Shoreham Nuclear Power Station Unit #1										DOCKET NUMBER (2) 0 5 0 0 0 3 2 2 1 OF 0 2										PAGE (3) 1 OF 0 2			
TITLE (4) "B" RBSVS initiation due to Technician error																							
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)									
1	0	0	9	8	5	8	5	0	4	8	0	0	1	1	0	8	8	5	0 5 0 0 0				
OPERATING MODE (9) 4			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)																				
POWER LEVEL (10) 0 1 0 0			20.402(b)			20.406(c)			<input checked="" type="checkbox"/> 50.73(a)(2)(iv)			73.71(b)											
			20.406(a)(1)(i)			50.36(a)(1)			<input type="checkbox"/> 50.73(a)(2)(v)			73.71(c)											
			20.406(a)(1)(ii)			50.36(a)(2)			<input type="checkbox"/> 50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)											
			20.406(a)(1)(iii)			50.73(a)(2)(i)			<input type="checkbox"/> 50.73(a)(2)(viii)(A)														
			20.406(a)(1)(iv)			50.73(a)(2)(ii)			<input type="checkbox"/> 50.73(a)(2)(viii)(B)														
			20.406(a)(1)(v)			50.73(a)(2)(iii)			<input type="checkbox"/> 50.73(a)(2)(ix)														
LICENSEE CONTACT FOR THIS LER (12)																							
NAME Robert W. Grunseich, Operational Compliance Engineer										TELEPHONE NUMBER 5 1 6 9 2 9 - 8 3 0 0													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																							
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS													
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR							
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO											

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single space typewritten lines) (16)

On October 9, 1985 at 1605, a Reactor Building Standby Ventilation System (RBSVS) "B" side initiation occurred due to an Instrument and Controls (I&C) technician error. The plant was in Operational Condition 4 with the mode switch in shutdown and the RPV temperature at 145 degrees F. The technician, who was fully qualified, was opening a circuit breaker in order to de-energize the circuit to a pressure switch in the RBSVS Chilled water system. The pressure switch was to be replaced under an approved Maintenance Work Request (MWR 85-5848). The circuit breaker that he opened supplied power to a subfeed panel to the RBSVS inlet and outlet dampers (1T46*AOV-035B & 037B). Opening the breaker resulted in a loss of power to the AOVs causing them to close. Closing the dampers resulted in a "B" side RBSVS initiation. The breaker was returned to the closed position, allowing the RBSVS "B" side to be returned to normal. Plant Management was notified of the event and the NRC was notified at 1750 per 10CFR50.72. To prevent recurrence, the I&C Engineer discussed the incident with the technician involved regarding the proper way to deenergize electrical circuits. In addition, a letter of reprimand was issued to the technician.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1):	DOCKET NUMBER (2):	LER NUMBER (8):			PAGE (3):		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Shoreham Nuclear Power Station Unit #1	0500032285	04	8	00	02	OF 02	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On October 9, 1985 at 1605 an "B" RBSVS initiation occurred due to a technician error. The plant was in Operational Condition 4 with the mode switch in shutdown and the RPV temperature at 145 degrees F.

The technician, who is fully qualified, was establishing conditions to support an approved modification to replace pressure switch 1M50*PS-029B under an approved Maintenance Work Request (MWR 85-5848). This pressure switch, provides a signal to isolate (close) the Chilled Water System Crosstie Supply Bypass Valve (1M50*ADV-069) on low system pressure. A Station Equipment Clearance Permit (SECP) was obtained to tag out 1M50*ADV-069B. However, this did not remove power from 1M50*PS-029B. The technician determined that Breaker #12, in 1R35-PNL-B1, controlled the power to the pressure switch. Upon inspection of Breaker #12 he found the breaker to be closed and opened it to deenergize the pressure switch. However, he did not completely investigate the consequences of pulling the breaker and the other components that were being fed from Breaker #12. This breaker, in panel 1R35-PNL-B1, is a supply breaker to 1H11-PNL-VC2 which is a sub-feed panel to 1T46*ADV-035B and 037B (RBSVS inlet and outlet dampers). When power is lost to these dampers, they will isolate (close). An auto initiation signal is then generated when either ADV is less than fully open, causing the "B" side RBSVS initiation. Breaker #12 in panel 1R-PNL-B1 was returned to the closed position and the RBSVS was returned to normal.

There was no safety significance to the event. All plant systems operated as designed. No ECCS systems were challenged or required for the event. Operators carried out all required actions. Plant Management was notified of the event and the NRC was notified at 1750 per 10CFR50.72.

The technician should have discussed the SECP with control room personnel and arranged for the proper de-energization of 1M50*PS-029B. To prevent recurrence, the I&C Engineer discussed the incident with the technician involved on the proper way to deenergize electrical circuits. A letter of reprimand was issued to the technician. In addition, the incident was reviewed in a meeting with all I&C technicians.



LONG ISLAND LIGHTING COMPANY

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TEL. (516) 929-8300

November 8, 1985

PM-85-257

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

In accordance with 10CFR50.73, enclosed is a copy of Shoreham Nuclear Power Station Unit #1's Licensee Event Report 85-048.

Sincerely yours,

William E. Steiger, Jr.
Plant Manager

WES/gr

Enclosure

cc: Dr. Thomas E. Murley, Regional Administrator
John Berry, Senior Resident Inspector
Institute of Nuclear Power Operations, Records Center
American Nuclear Insurers

SR.A21.0200

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11