

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) RANCHO SECO NUCLEAR GENERATING STATION UNIT NO. 1	DOCKET NUMBER (2) 0 5 0 0 0 3 1 1 2	PAGE (3) 1 OF 0 1 2
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TITLE (4)

## INVALIDATION OF APPENDIX R ANALYSIS VIA OPEN PZR SAMPLE VALVE

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)					
1	1	2	0	8	5	8	5	0	2	2	0	5	0	0	0
1	1	2	0	8	5	8	5	0	2	2	0	5	0	0	0

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																			
	20.402(b)					20.405(c)					50.73(a)(2)(iv)					73.71(b)				
	20.405(a)(1)(i)					50.36(c)(1)					50.73(a)(2)(v)					73.71(c)				
	20.405(a)(1)(ii)					50.36(c)(2)					50.73(a)(2)(vii)					OTHER (Specify in Abstract below and in Text, NRC Form 366A)				
	20.405(a)(1)(iii)					50.73(a)(2)(i)					50.73(a)(2)(viii)(A)									
	20.405(a)(1)(iv)					50.73(a)(2)(ii)					50.73(a)(2)(viii)(B)									
20.405(a)(1)(v)					50.73(a)(2)(iii)					50.73(a)(2)(ix)										

## LICENSEE CONTACT FOR THIS LER (12)

NAME		TELEPHONE NUMBER	
Ron W. Colombo, Regulatory Compliance Superintendent		AREA CODE	

## COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

## SUPPLEMENTAL REPORT EXPECTED (14)

<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
			0	3	1

## ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On November 20, 1985, it was reported via an internal Occurrence Description Report, that pressurizer sampling valve (HV-21516) was found in the open position with its breaker (2B203) racked out. This configuration violated two (2) caution tags which identify HV-21516 as a 10 CFR 50, Appendix R controlled valve required to be closed with its breaker racked out. The discrepancy was detected at 0218 hours on November 20, 1985, when operations personnel, preparing to take a sample, racked in the breaker and found the valve open. By checking the previous log entry for HV-21516, it was determined that the valve had been open since 1510 hours on November 19, 1985 (approximately 11 hours). As corrective actions, HV-21516 was closed and breaker 2B203 racked out following the sampling activity. Both actions were dual verified by operations personnel. This configuration invalidates certain valve position assumptions made in the District's 10 CFR 50, Appendix R analysis for achieving hot shutdown from the shutdown panel following a fire in the Control Room. The event is being reported in accordance with 10 CFR 50.73(a)(2)(v).

The District's Incident Analysis Group will perform a root cause analysis of the event to determine what further corrective action is required to prevent a recurrence. Corrective action resulting from this analysis will be reported in a supplement to this report.

There were no effects on plant or public safety as a result of this event.

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PDR ADDOCK 05000312  
S PDR

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)  RANCHO SECO NUCLEAR GENERATING STATION UNIT NO. 1	DOCKET NUMBER (2)  0   5   0   0   0   3   1   2   8   5   —   0   2   2   —   0   0   0   2   OF   0   2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On November 20, 1985, it was reported via an internal Occurrence Description Report, that pressurizer sampling valve (HV-21516) was found in the open position with its breaker (2B203) racked out. This configuration invalidates certain valve position assumptions made in the District's 10 CFR 50, Appendix R analysis for achieving hot shutdown from the shutdown panel following a fire in the Control Room. The as-found configuration, if combined with a "failed open" pressurizer sample isolation valve (SFV-70003), could result in an uncontrolled reactor coolant system fluid loss and compromise the plant's capability to achieve hot shutdown from the panel.

The discrepancy was detected at 0218 hours on November 20, 1985, when operations personnel, preparing to take a sample, racked in the breaker and found HV-21516 open. By checking the previous log entry for the valve, it was determined that the valve had been open since 1510 hours on November 19, 1985 (approximately 11 hours). During this period, the reactor power level was 83% of full power.

Following the sampling which led to the detection of the improper configuration, operations personnel took the corrective action of restoring the proper configuration by: (1) closing HV-21516 and (2) racking out breaker 2B203. Both actions were dual verified.

In reaching the reported improper configuration, two CAUTION tags (#570 and 602) were violated. These tags identify HV-21516 as a 10 CFR 50, Appendix R controlled valve required to be closed with the breaker racked out. Any deviation from these conditions requires the permission of the Control Room shift supervisor. In addition, procedure SP 214.03 (Locked Valve List), which provides for administrative control of HV-21516, was also violated.

The District's Incident Analysis Group will perform a root cause analysis of the event to determine what further corrective action is required to prevent a recurrence of the event. Corrective action resulting from this analysis will be reported in a supplement to this report.

There were no effects on plant or public safety as a result of this event.



**SMUD**

SACRAMENTO MUNICIPAL UTILITY DISTRICT □ 6201 S Street, P.O. Box 15830, Sacramento CA 95852-1830, (916) 452-3211  
AN ELECTRIC SYSTEM SERVING THE HEART OF CALIFORNIA

RJR 85-589

December 18, 1985

J B MARTIN REGIONAL ADMINISTRATOR  
REGION V OFFICE OF INSPECTION AND ENFORCEMENT  
ATTN DOCUMENT CONTROL DESK  
U S NUCLEAR REGULATORY COMMISSION  
WASHINGTON DC 20555

DOCKET NO. 50-312  
LICENSE NO. DPR-54  
LICENSEE EVENT REPORT NUMBER 85-22

In accordance with the requirements of 10 CFR 50.73(a)(2)(v), the Sacramento Municipal Utility District hereby submits Licensee Event Report Number 85-22.

If there are any questions concerning this report, please contact Mr. Ron Colombo at the Rancho Seco Nuclear Generating Station Unit No. 1.

R. J. RODRIGUEZ  
ASSISTANT GENERAL MANAGER,  
NUCLEAR

Attachment

cc: Region V (2)  
INPO

IE22  
1/1