

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Callaway Plant Unit 1 DOCKET NUMBER (2) 0 5 0 0 0 4 8 3 1 OF 0 2

TITLE (4) Inadvertent Engineered Safety Features Actuation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)	
1	1	27	85	05	00	1	22	78		0 5 0 0 0	
										0 5 0 0 0	

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
POWER LEVEL (10)	1 0 0	20.402(b)	20.405(c)	X	50.73(a)(2)(iv)	73.71(b)					
		20.405(a)(1)(i)	50.38(c)(1)		50.73(a)(2)(v)	73.71(c)					
		20.405(a)(1)(ii)	50.23(c)(2)		50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)					
		20.405(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)						
		20.405(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)						
		20.405(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(x)						

LICENSEE CONTACT FOR THIS LER (12)
NAME Charles D. Naslund - Superintendent, I&C TELEPHONE NUMBER 3 1 4 6 7 6 - 8 5 0 0
AREA CODE

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	
X	J E	F U	G O 6 3	Y						

SUPPLEMENTAL REPORT EXPECTED (14)
YES (If yes, complete EXPECTED SUBMISSION DATE) X NO
EXPECTED SUBMISSION DATE (15)

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 11/27/85 at 1219 CST a Control Room Ventilation Isolation and a Containment Purge Isolation Signal occurred as a result of a failed fuse at the microprocessor for Containment Purge Radiation Monitor GT-RE-33 and a tripped breaker supplying power to the flow pump for GT-RE-33. At the time of the incident, the plant was in Mode 1. Power Operations, at 100% power and normal operating temperature and pressure.

An investigation did not identify any problems with the power feeds to GT-RE-33 or GT-RE-33 itself. The fuse was replaced, the breaker closed, and power restored at approximately 1415 CST with no additional problems. This incident is therefore considered an isolated case for which no further corrective action is required.

There was no damage to plant equipment or release of radioactivity as a result of this event. Purging of containment was not in progress at the time of this event and the redundant Containment Purge Radiation Monitor, GT-RE-22, was operable. This event was not caused by actual radiation levels, therefore, the public health and safety was not endangered.

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PDR ADOCK 05000483
S PDR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1) Callaway Plant Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 4 8 3 8 5 - 0 5 0 - 0 0 0 2	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 11/27/85 at 1219 CST, a Control Room Ventilation Isolation (CRVIS) and a Containment Purge Isolation Signal (CPIS) were received. The alarm printer (RM-11) indicated that the radiation levels had not exceeded the setpoint. I&C technicians were called in to investigate the event. They found that the CRVIS and the CPIS were generated by a downscale trip on Containment Purge Radiation Monitor GT-RE-33⁽¹⁾ due to a loss of power. The loss of power was due to a tripped supply breaker which supplies power to GT-RE-33's pump and a failed fuse⁽²⁾ at the RM-80 microprocessing unit. The required Engineered Safety Features equipment functioned as designed.

Troubleshooting of the event included isolating and testing the two power feeds that lead into the Containment Purge Radiation Monitor GT-RE-33 to see if wiring between the two could have caused the problem. This did not identify any problems. The failed fuse was replaced, the supply breaker was reset, and power was restored to GT-RE-33 at approximately 1415 CST with no problems resulting. Also, plant personnel working in the area near GT-RE-33 were questioned to see if any unusual activities were going on at the time of the trip. This did not identify any problems. Since no further equipment problems were identified, this event is considered an isolated case for which no further corrective action is deemed necessary.

There was no damage to plant equipment or release of radioactivity as a result of this event. Purging of containment was not in progress at the time of this event and the redundant Containment Purge Radiation Monitor, GT-RE-22, was operable. This event was not initiated by actual radiation levels, therefore the public health and safety was not threatened at any time during the course of this event.

Previous occurrences: none

Footnotes

- (1) IEEE Standard 805-1983 System - IL
IEEE Standard 803A-1983 Component - RI
- (2) IEEE Standard 805-1983 System - IL
IEEE Standard 803A-1983 Component - FU
Manufacturer - GA Technologies
Model No. 50008728-001



UNION ELECTRIC COMPANY

Callaway Plant

December 20, 1985

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

ULNRC-1229

Gentlemen:

DOCKET NUMBER 50-483
CALLAWAY PLANT UNIT 1
FACILITY OPERATING LICENSE NPF-30
LICENSEE EVENT REPORT 85-050-00
INADVERTENT ENGINEERED SAFETY FEATURES ACTUATION

The enclosed Licensee Event Report is submitted pursuant to
10 CFR 50.73(a)(2)(iv) concerning an unplanned Control Room
Ventilation Isolation.

S. E. Miltenberger
S. E. Miltenberger
Manager, Callaway Plant

can WRB dls
CDN/WRB/drs
Enclosure

cc: Distribution attached

IE22
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cc distribution for ULNRC-1229

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3456-0260
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N. Date