

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)  
Cooper Nuclear Station

DOCKET NUMBER (2)

0 5 0 0 0 2 9 8 1 OF 0 4

TITLE (4)  
Inoperable Snubbers Due to Inadequate Inspection ProcedureEVENT DATE (5)  
MONTH DAY YEAR  
1 1 1 9 8 5  
LER NUMBER (6)  
YEAR SEQUENTIAL NUMBER REVISION NUMBER  
8 5 - 0 1 6 - 0 0 1  
REPORT DATE (7)  
MONTH DAY YEAR  
1 2 1 9 8 5  
OTHER FACILITIES INVOLVED (8)  
FACILITY NAMES  
DOCKET NUMBER(S)  
0 5 0 0 0  
0 5 0 0 0OPERATING MODE (9)  
N  
POWER LEVEL (10)  
0 1 0 1 0  
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8. (Check one or more of the following) (11)  
20.402(b) 20.405(c) 50.73(a)(2)(iv) 73.71(b)  
20.405(a)(1)(i) 50.38(c)(1) 50.73(a)(2)(v) 73.71(c)  
20.405(a)(1)(ii) 50.35(c)(2) 50.73(a)(2)(vii) X OTHER (Specify in Abstract below and in Text, NRC Form 366A)  
20.405(a)(1)(iii) 50.73(a)(2)(i) 50.73(a)(2)(viii)(A)  
20.405(a)(1)(iv) 50.73(a)(2)(ii) 50.73(a)(2)(viii)(B)  
20.405(a)(1)(v) 50.73(a)(2)(iii) 50.73(a)(2)(ix)LICENSEE CONTACT FOR THIS LER (12)  
NAME  
E. M. Mace, Plant Engineering Supervisor  
TELEPHONE NUMBER  
AREA CODE  
4 0 2 8 2 5 1 - 3 8 1 1COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)  
CAUSE SYSTEM COMPONENT MANUFACTURER REPORTABLE TO NPDOS  
D B I O S I P T I 1 2 0 1 7 Y  
D S I B S I P T I 1 2 0 1 7 Y  
D B I O S I P T I 1 2 0 1 7 Y  
D B I N S I P T I 1 2 0 1 7 YSUPPLEMENTAL REPORT EXPECTED (14)  
YES (If yes, complete EXPECTED SUBMISSION DATE) NO  
X  
EXPECTED SUBMISSION DATE (15)  
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On November 19, 1985, discrepancies were noted on 43 of 290 seismic support (snubbers) during a 100% visual inspection of plant safety related snubbers. Of the 43, 21 were declared inoperable per station Technical Specification visual inspection acceptance criteria. The reactor was in cold shutdown (i.e., coolant temperature less than 212 degrees F and the reactor vented) for repairs to the main turbine generator at the time of this event.

The 100% visual inspection of plant safety related snubber systems was initiated as a result of the observations from an inspection tour by regional Nuclear Regulatory Commission (NRC) personnel. The findings indicated weaknesses in the snubber inspection program at Cooper Nuclear Station. Specifically, procedural and training discrepancies relating to the visual inspection of snubber "systems". The "system" refers to the snubber and associated attachments such as pipe clamps, paddle-clevis arrangements and external interferences. The inoperable snubber systems were repaired and returned to an operable status prior to reactor startup. This event had no impact to public health and safety.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)  Cooper Nuclear Station	DOCKET NUMBER (2)  0 5 0 0 0 2 9 8	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	0 1 6	0 0	0 2	OF	0 4

TEXT (If more space is required, use additional NRC Form 365A's) (17)

On October 22, 1985, while on a tour of the primary containment, resident and regional NRC personnel noted a mechanical snubber being restricted in movement by floor grating. After the problem was brought to the attention of station management, the subject seismic support was declared inoperable and a repair program was initiated. Other selected snubbers were visually inspected for external interferences in accordance with station Technical Specification visual inspection criteria. The inspection was later expanded to cover all safety-related snubbers in the primary containment. As a result of this inspection, three additional snubbers were found with interference problems and were subsequently declared inoperable. Modifications were performed on the seismic supports to return them to operable status.

On November 19, 1985, a follow-up meeting was held between station management and the NRC inspectors to discuss the corrective actions taken for the October 22, 1985 event. Attendees felt that adequate attention was given to snubbers inside primary containment, but possibly not enough attention had been given to safety-related snubbers outside primary containment. Therefore, ten snubbers were randomly selected outside containment for visual inspection. During the course of this inspection, several additional seismic supports were found to have discrepancies. The inspection results indicated a significant weakness in the snubber inspection program, specifically procedural and training deficiencies related to visual inspection of snubber "systems". The "system" refers to the snubber and associated attachments such as pipe clamps, paddle-clevis arrangements and external interferences.

Additional visual inspections were initiated using existing station inspection procedures which were supplemented to include added inspection criteria in the areas of proper snubber alignment, correct installation and proper clearances. These additional visual inspections included all safety-related snubbers outside primary containment and 10% of safety-related snubbers inside containment (100% of snubbers inside containment had been previously inspected for external interferences only). The 10% inspection of snubbers inside containment was later expanded to cover all safety-related snubbers inside containment. A total of 43 out of 290 seismic supports were found with discrepancies. Out of the 43, 21 were declared inoperable per station Technical Specification visual inspection acceptance criteria. The following lists the component identification code (CIC) of the inoperable seismic supports and the discrepancy found.

CIC	Discrepancy
1) RHR-Snub-(RH-S34)	Pipe Clamp Rotated
2) RHR-Snub-(RH-S36)	Pipe Clamp Rotated
3) MS-Snub-(MS-S14)	Pipe Clamp Rotated
4) RCIC-Snub-(RF-S1)	Pipe Clamp Rotated
5) RHR-Snub-(RH-S54)	Pipe Clamp Rotated
6) RHR-Snub-(RH-S29)	Pipe Clamp Rotated
7) RWCU-Snub-(CU-S89)	Pipe Clamp Rotated
8) RHR-Snub-(RH-S32)	Pipe Clamp Rotated
9) MS-Snub-(MS-S11)	Pipe Clamp Rotated
10) MS-Snub-(MS-S11A)	Pipe Clamp Rotated

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/86

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Cooper Nuclear Station	05000298885	—	016	—	00	03	OF 04

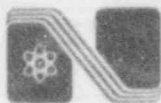
TEXT (If more space is required, use additional NRC Form 366A's) (17)

- |                         |                                             |
|-------------------------|---------------------------------------------|
| 11) RF-Snub-(RF-S8)     | Pipe Clamp Rotated                          |
| 12) MS-Snub-(VR-59-7-X) | Pipe Clamp Rotated                          |
| 13) MS-Snub-(VR-S63)    | Pipe Clamp Rotated                          |
| 14) MS-Snub-(VR-S4)     | Pipe Clamp Rotated                          |
| 15) MS-Snub-(VR-55-9-2) | Pipe Clamp Rotated                          |
| 16) MS-Snub-(BS-S116A)  | Pipe Clamp Rotated, Rear Bracket Misaligned |
| 17) MS-Snub-(MS-S15A)   | Anchor Bolts Loose, Pipe Clamp Deformed     |
| 18) RF-Snub-(RF-S6)     | Pipe Clamp Loose                            |
| 19) RHR-Snub-(RH-S27A)  | Anchor Bolts Loose, Snubber Paddle Bent     |
| 20) MS-Snub-(MS-S13A)   | Snubber Paddle Bent                         |
| 21) RHR-Snub-(RH-S41)   | Pipe Clamp Loose                            |

No generic failure is attributed to the type, model, size or manufacture of any snubber, but rather a deficiency in the visual inspection procedures and training relating to the snubber inspection program. The 21 inoperable seismic supports were repaired and returned to operable status prior to reactor startup.

It should be noted that because the reactor was in a cold shutdown at the time of this event, no Technical Specifications were violated. However, it is felt that this event would be of particular interest to other licensees. This event presented no adverse consequences to public health and safety.

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)



## Nebraska Public Power District

COOPER NUCLEAR STATION  
P.O. BOX 98, BROWNVILLE, NEBRASKA 68321  
TELEPHONE (402) 825-3811

CNSS850708

December 19, 1985

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Dear Sir:

Cooper Nuclear Station Licensee Event Report No. 85-016 is forwarded as an attachment to this letter.

Sincerely,

P. V. Thomason  
Division Manager of  
Nuclear Operations

PVT:lb

cc: R. D. Martin  
L. G. Kunc1  
J. D. Weaver  
L. R. Berry  
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