

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)

River Bend Station, Unit 1

DOCKET NUMBER (2)

0 5 0 0 3 4 5 8 1 OF 0 3

PAGE (3)

TITLE (4)

Reactor Water Cleanup Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MO	TH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)											
1	1	1	5	8	5	0	3	7	0	0	1	2	1	7	8	5	0	5	0	0	0	0
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																						
OPERATING MODE (9)			20.402(b)			20.408(a)			X			80.73(a)(2)(iv)			73.71(b)							
POWER LEVEL (10)			0 0 1			20.408(a)(1)(i)			80.38(a)(1)			80.73(a)(2)(v)			73.71(a)							
			20.408(a)(1)(ii)			80.38(a)(2)			80.73(a)(2)(vi)			80.73(a)(2)(vii)(A)			OTHER (Specify in Abstract below and in Text, NRC Form 308A)							
			20.408(a)(1)(iii)			80.73(a)(2)(i)			80.73(a)(2)(viii)(A)			80.73(a)(2)(viii)(B)										
			20.408(a)(1)(iv)			80.73(a)(2)(ii)			80.73(a)(2)(ix)			80.73(a)(2)(ix)										
			20.408(a)(1)(v)			80.73(a)(2)(iii)			80.73(a)(2)(x)			80.73(a)(2)(x)										

LICENSEE CONTACT FOR THIS LER (12)

NAME

R. G. West

TELEPHONE NUMBER

AREA CODE

5 0 4 3 4 6 1 - 8 6 1 5 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC			
B	I	J	T	A	1	G	I	O	8	1	0	Y
B	I	J	T	D	S	P	O	5	1	5	Y	

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On 11/17/85 at 2020, with the unit in operational condition 2 (startup), an erroneous Reactor Water Cleanup (RWC) isolation occurred. The isolation was caused by a defective trip unit.

The trip condition was reset and the RWC system was returned to service with no further problems.

There was no impact on the safe operation of the plant or the health and safety of the public.

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PDR ADOCK 05000458
S PDR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 306A's) (17)

On 11/17/85 at 2020, with the unit in operational condition 2 (startup) and reactor power at thirty (30) MWT, a Reactor Water Cleanup (RWCU) (IEEE:CF) isolation occurred. The isolation occurred while maintenance personnel were investigating the cause for an erroneous alarm.

The Control Operating Foreman (COF), during his rounds, noted that an RWCU demineralizer backwash tank area high temperature alarm was present without an isolation. The Maintenance Department was immediately contacted and an Instrument and Controls Technician was dispatched to investigate the annunciator circuitry. The isolation occurred when the technician opened a temperature switch cover and inadvertently flipped temperature switch (E31-N627B) to the read position.

The temperature switch which caused the isolation is manufactured by U.S. Riley Corporation. Normally, placing the switch to the read position would not cause an isolation. However, this type of temperature switch (model 86V and 86 Temp-Matic) has been identified as causing a spurious isolation when activating the read function (reference LERs 85-009, 029, 031 and 035). To prevent this from occurring a design change (MR-589, MWR 15553) has been issued but had not been implemented for this device at the time of the event. The trip condition was immediately reset and the RWCU system was returned to service.

Maintenance later determined that a defective isolator card (E31A-AT5-0) was the cause for the erroneous high temperature alarm. The isolator card was replaced and no further problems occurred.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 306A's) (17)

There was no impact on the safe operation of the plant or the health and safety of the public.



GULF STATES UTILITIES COMPANY

RIVER BEND STATION POST OFFICE BOX 220 ST. FRANCISVILLE, LOUISIANA 70775
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December 17, 1985
RBG-22851
File Nos. G9.5, G9.25.1.3

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

River Bend Station - Unit 1
Docket No. 50-458

Please find enclosed Licensee Event Report No. 85-037 for River Bend Station - Unit 1. This report is submitted pursuant to 10CFR50.73.

Sincerely,

J. E. Booker
Manager-Engineering,
Nuclear Fuels & Licensing
River Bend Nuclear Group

JEB/TFP/PDG/BEH/ebm

cc: U.S. Nuclear Regulatory Commission
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