CONNECTICUT YANKEE ATOMIC POWER COMPANY

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December 11, 1985

Docket No. 50-213
B11911

Director of Nuclear Reactor Regulation
Attn: Mr. Christopher I. Grimes, Director
Integrated Safety Assessment Project
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Gentlemen:

Haddam Neck Plant
Proposed Revision to Technical Specifications
Cycle 14 Reload

Pursuant to 10 CFR 50.90, Connecticut Yankee Atomic Power Company (CYAPCO) hereby proposes to amend Operating License No. DPR-61 for the Haddam Neck Plant by incorporating the attached changes into the plant Technical Specifications. The proposed changes revise Technical Specifications 1.0, 2.4, 3.16, 3.17, 3.18 and 3.20 for Cycle 14 operation. Revised pages are shown in Attachment 1 and detailed descriptions are provided for each change in Attachment 2.

CYAPCO has reviewed the attached proposed changes pursuant to the requirements of 10 CFR 50.59 and has determined that they do not constitute an unreviewed safety question. The basis for this determination is discussed in Attachment 3.

CYAPCO has reviewed the proposed changes, in accordance with 10 CFR 50.92 and has concluded that they do not involve a significant hazards consideration. The basis for this conclusion is that the three criteria of 10 CFR 50.92(c) are not compromised, a conclusion which is supported by our determination made pursuant to 10 CFR 50.59. The Commission has provided guidance concerning the application of the standards in 10 CFR 50.92 by providing certain examples (48 FR 14870, April 6, 1983) of amendments that are considered not likely to involve significant hazards considerations. Example (iii) relates to a change resulting from a nuclear reactor core reloading, if no fuel assemblies significantly different from those found previously acceptable to the NRC for a previous core at the facility in question are involved. This assumes that no significant changes are made to the acceptance criteria for the technical specifications, that the analytical methods used to demonstrate conformance with the technical specifications and regulations are not significantly changed, and that the NRC has previously found such methods acceptable. The attached proposed changes for Cycle 14 are similar to example (iii) in that they involve



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changes resulting from a nuclear reactor core reloading. The fuel assemblies are identical in design to those found previously acceptable to the NRC for Cycle 13. No significant changes have been made to the acceptance criteria for the Technical Specifications, and the analytical methods used to demonstrate conformance with the Technical Specifications and regulations are not significantly changed in Cycle 14 from those which the NRC has previously found acceptable in Cycle 13. CYAPCO considers Cycle 14 to be an equilibrium fuel cycle that does not require a revision to the design basis accident analysis. Attachment 3 provides a detailed description of the effect of these changes on the design basis accident analysis. A summary of each change is provided below.

The change to the definition of Quadrant Power Tilt Ratio is consistent with the previously approved Westinghouse Standard Technical Specifications (STS). The change to the Variable Low Pressure Trip Setpoint provides the same margin to the DNB limit as the present specification. The change to the Tilt Limit is consistent with the previously approved Westinghouse STS. The change to the Nuclear Overpower Trip Setpoint does not change the maximum allowable reactor power. The additional restrictions on the Moderator Temperature Coefficient are in accordance with the previously approved LOCA analysis. The change to the Linear Heat Generation Rate Limits is to account for the effects of plugging/sleeving steam generator tubes on the previously approved large break LOCA analysis. The change to the Axial Offset Limits is consistent with the previously approved STS reload analysis methodology. The additional Technical Specification on Minimum Reactor Coolant System Flowrate is consistent with the previously approved safety analysis and the Standard Review Plan acceptance criteria. In summary, CYAPCO concludes that these changes do not involve a significant hazards consideration.

The results of the LOCA calculations will be submitted by January 1, 1986 along with the steam generator tube sleeving qualification report. These submittals will provide further analytical bases for the above conclusions. The Haddam Neck Nuclear Review Board has reviewed and approved this proposed amendment and has concurred with the above determinations.

In accordance with 10 CFR 50.91(b), CYAPCO is providing the State of Connecticut with a copy of this proposed amendment.

Pursuant to the requirements of 10 CFR 170.12(c), enclosed with this amendment request is the application fee of $150.00.

We trust you find this information satisfactory and request review and approval of this amendment request by March 1, 1986 in order to support the start of Cycle 14.

Very truly yours,

CONNECTICUT YANKEE ATOMIC POWER COMPANY

J. F. Opeka
Senior Vice President

cc: Kevin McCarthy
T. E. Murley

STATE OF CONNECTICUT )
) ss. Berlin
COUNTY OF HARTFORD )

Then personally appeared before me J. F. Opeka, who being duly sworn, did state that he is Senior Vice President of Connecticut Yankee Atomic Power Company, a Licensee herein, that he is authorized to execute and file the foregoing information in the name and on behalf of the Licensees herein and that the statements contained in said information are true and correct to the best of his knowledge and belief.

Lorraine J. D'Amico
Notary Public

My Commission Expires March 31, 1988