NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Indian Point, Unit 3	0 5 0 0 0 2 8 6	1 OF 0 2

TITLE (4)
Unit Trip Initiated by Malfunctioning Feedwater Regulating Valve

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
1 0	0 4	8 5	8 5	— 0 0 6	— 0 0	1 1 0	8 5			0 5 0 0 0

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 2 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
John J. Anderson , Assistant Plant Engineer	9 1 4 — 7 3 9 - 8 2 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	S J	F C V	B 0 4 5	Y							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On October 4, 1985 during initial unit startup from a scheduled refueling outage, a turbine trip and subsequent reactor trip were initiated automatically by a high level in No. 33 steam generator. Reactor power was 20 percent at the time of the trip. The high water level was caused by sluggish response of No. 33 main feedwater regulating valve as the operator switched from low feedwater flow regulation to main feedwater flow regulation. The irregular response was found to have been caused by a malfunction of the valve's positioner linkage assembly. The assembly was replaced, and the other main feedwater regulators were inspected. The unit was subsequently synchronized to the bus.

* U.S.GPO:1984-0-454-481/18759

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
- Indian Point, Unit 3	0 5 0 0 0 2 8 6	8 5	— 0 0 6	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 1810 hours on October 4, 1985, a turbine trip and subsequent reactor trip were initiated automatically by a high water level in No. 33 steam generator. The unit had just returned to service from a scheduled refueling outage. Reactor power was 20 percent at the time of the trip. No. 33 main feedwater (IEEE Code SJ) regulating valve (FRV) responded sluggishly to manual control signals as the control room operators switched from low feedwater flow regulation to main feedwater flow regulation. The resulting feedwater perturbations caused the water level in No. 33 steam generator to reach the high level trip setpoint.

Investigation determined that the positioner linkage assembly made by Bailey Meter Co. (Code B045) on No. 33 FRV had overspanned its full travel limit. The sluggish valve response was due to the fact that this linkage provides a valve position feedback function. Since all four FRV's were inspected and stroked before power ascension began, it is believed that the linkage malfunction occurred during manual operation of the valve prior to the trip.

Automatic closure of No. 33 FRV after the trip caused the overspanned linkage to bend. The positioner assembly was entirely replaced. Close inspection of the remaining linkage assemblies revealed no abnormalities. All four FRV's were cycled to verify operability before startup commenced. The unit was synchronized to the bus at 2323 hours on October 4, 1985.

All equipment associated with the reactor trip performed its designated function. A similar event occurred on January 23, 1985 (LER 85-002-00).

Indian Point 3
Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511
914 739.8200

October 31, 1985
FWG-IP3-2671

Docket No. 50-286
License No. DPR-64

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sir:

The attached Licensee Event Report LER 85-006-00 is hereby submitted in accordance with the requirements of 10CFR50.73. This event is of the type defined in Paragraph 50.73 (a) (2) (iv).
Very truly yours,

William A. Josiger
Resident Manager

FWG:aa:08
Attachment

cc: Dr. Thomas Murley
Regional Administrator
Region 1
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

IP3 Resident Inspectors' Office
J. P. Bayne, WPO
J. C. Brons, WPO
G. M. Wilverding (SRC), WPO

INPO Records Center
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

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