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October 31 1985
RFW - 0670

Mr. John A. Zwolinski, Chief
Operating Reactors Branch No. 5
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Zwolinski:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Suppression Pool Local Temperature Limit

- References: 1) NRC Order dated January 19, 1982
- 2) Letter, D. M. Crutchfield to P. B. Fiedler,
"Mark I Containment Long Term Program - Structural
Review", January 13, 1984
- 3) Letter, J. M. Fulton to C. O. Thomas,
"Request for Review of NEDO 30832", March 21, 1985

In our letter dated July 26, 1985, concerning the work scope for our next (Cycle 11) refueling outage, we listed eight items in Attachment 2 for which we proposed a scope change or cancellation. Based upon the technical bases presented in NEDO 30832 and our previous analysis of Electromatic Relief Valve discharges at Oyster Creek, we conclude that it is not necessary to provide suppression pool temperature monitoring local to the quenchers (Item 7) or the thermal mixing modification for the suppression pool (Item 6). Our previously proposed modifications for suppression pool local temperature monitoring and thermal mixing had been in response to NUREGs 0661 and 0783.

GPU Nuclear endorses the BWR Owner's Group position contained in the General Electric (GE) report NEDO 30832, "Elimination of Limit on BWR Suppression Pool Temperature for SRV Discharge with Quenchers", December 1984. The GE report concluded that unstable steam condensation is not a concern when quencher devices are utilized on SRV discharge piping. Quenchers have been installed at Oyster Creek. NRC review of the NEDO report was requested by Reference 3. Our review of the report concluded that it is applicable to the Oyster Creek Nuclear Generating Station.

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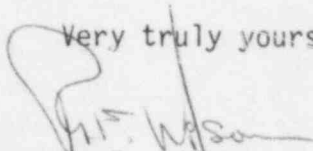
Pool
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The issue of suppression pool temperature limits was addressed at a September, 1983 meeting with Brookhaven National Laboratory (BNL). Based upon BNL's review of GPU Nuclear Technical Data Report No. 187, Revision 1, "Report on the Effects of Electromatic Relief Valve Discharge on Torus Water Temperature", coupled with the Monticello suppression pool temperature data base, General Electric Report No. NDE-24542-P, April 1979, BNL concluded that "...local pool temperature will not exceed the limits dictated by the acceptance criteria during the most severe SRV transients of interest". BNL Report No. 04243, "Technical Evaluation of the Oyster Creek Plant Unique Analysis Reports", was forwarded to us by Reference 2.

We request NRC concurrence with our cancellation of the modifications discussed above with the understanding that they will no longer be considered as part of the scope of work required by the Order (Reference 1) relating to the Mark I Containment Long Term Program.

If you should have any questions concerning the above matter, please contact Mr. Michael W. Laggart, Manager BWR Licensing, at (201) 299-2341.

Very truly yours,


R. F. Wilson
Vice President and Director
Technical Functions

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