

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Brunswick Steam Electric Plant Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 2 5										PAGE (3) 1 OF 0				
TITLE (4) Primary Containment Groups 6 and 8 Isolations, Auto-Isolation of Reactor Building Ventilation System, and Auto-Start of Reactor Building Standby Gas Treatment System During Testing																								
EVENT DATE (5)					LER NUMBER (6)					REPORT DATE (7)					OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER			MONTH	DAY	YEAR			FACILITY NAMES					DOCKET NUMBER(S)				
10	01	85	85		05	3				00	10	25	85							0 5 0 0 0				
OPERATING MODE (9) 5					THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																			
POWER LEVEL (10) 0 0 0					20.402(b)					20.405(e)					X 50.73(a)(2)(iv)					73.71(b)				
					20.405(a)(1)(i)					50.38(a)(1)					50.73(a)(2)(v)					73.71(e)				
					20.405(a)(1)(ii)					50.38(a)(2)					50.73(a)(2)(vi)					OTHER (Specify in Abstract below and in Text, NRC Form 365A)				
					20.405(a)(1)(iii)					50.73(a)(2)(ii)					50.73(a)(2)(viii)(A)									
					20.405(a)(1)(iv)					50.73(a)(2)(iii)					50.73(a)(2)(viii)(B)									
20.405(a)(1)(v)					50.73(a)(2)(iii)					50.73(a)(2)(ix)														
LICENSEE CONTACT FOR THIS LER (12)																								
NAME M. J. Pastva, Jr., Regulatory Technician															TELEPHONE NUMBER 9 1 9 4 5 7 - 1 2 3 1 5									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																								
CAUSE	SYSTEM	COMPONENT	MANUFAC TURE	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFAC TURE	REPORTABLE TO NRC															
SUPPLEMENTAL REPORT EXPECTED (14)										YES (If yes, complete EXPECTED SUBMISSION DATE)					NO					EXPECTED SUBMISSION DATE (15)				

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

On 10/01/85, at 1404, Unit 1 primary containment Groups 6 and 8 isolations, auto-isolation of the Reactor Building ventilation system, and auto-start of the Standby Gas Treatment (SBGT) System occurred. At the time, test equipment was being connected for response time testing high drywell pressure instrumentation as part of a plant modification acceptance test. Unit 1 was in a refuel/maintenance outage.

This event was initiated by the incorrect connection of test jacks of the subject test equipment to a test equipment recorder. Electrical grounding of fuse A71B-F22 in the primary containment isolation circuitry consequently occurred and the event resulted. A contributory factor to this event was that the involved test procedure did not require verification of test connection or caution concerning the consequences of an incorrect test jack connection.

Within 45 minutes of this event, the subject fuse was replaced, the isolation and starting signals were reset, and the affected systems were returned to service.

The involved test procedure has been revised to incorporate a caution requiring correct setup and verification of test jacks prior to connection to logic circuitry.

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Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429
October 25, 1985

FILE: B09-13510C
SERIAL: BSEP/85-1864

NRC Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
LICENSEE EVENT REPORT 1-85-053

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

MJP/dj

Enclosure

cc: Dr. J. N. Grace

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