



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

SL-0299
PDR 113185

June 26, 1984

Honorable Nunzio J. Palladino
Chairman
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Dr. Palladino:

SUBJECT: TWO-HUNDRED NINETIETH MEETING OF THE ADVISORY COMMITTEE ON
REACTOR SAFEGUARDS, JUNE 14-16, 1984

290th ACRS Meeting

The Committee met with representatives of the NRC Staff, Pacific Gas and Electric Company (PG&E), the U.S. Geological Survey, and J. K. Crouch to review the proposed license condition that would require PG&E to do a seismic study to reassess the Diablo Canyon Nuclear Power Plant design bases and the appropriateness of the utility taking the lead in this project. The Committee also reviewed a recent technical paper by J. K. Crouch, S. Bachman, and J. Shay, and testimony given before the Commission on this paper.

The Committee met with representatives of the NRC Staff and completed its report to the NRC regarding the proposed NRC Safety Research Program Budget for FYs 1986 and 1987.

The Committee met with representatives of the NRC Staff, the affected users, and advocates of reduced availability of highly enriched uranium fuel to review a proposed rule for limiting the use of highly enriched uranium in research and test reactors and a plan to increase security measures at nonpower reactors that use high-enrichment fuel.

Reports on the subjects noted above have been sent to you.

The Committee briefly discussed Regulatory Guides 1.35, Revision 3, "Inservice Inspection of UngROUTed Tendons in Prestressed Concrete Containments" and 1.35.1, "Determining Prestressing Forces for Inspection of Prestressed Concrete Containments" and the issuance of the proposed General Revision to Appendix J to 10 CFR Part 50, "Leak Tests for Primary and Secondary Containments of Light-Water-Cooled Nuclear Power Plants," for public comment.

The Committee's comments on the subject noted above have been provided to the Executive Director for Operations. A copy has been sent to you.

The Committee requested information from the EDO regarding NRC Staff activities related to steam generator overfill in PWRs.

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The Committee discussed the NRC program for management of plant-specific backfitting of operating power reactors through the implementation of interim procedures and requested that the EDO provide an opportunity to review the proposed final NRC Manual Chapter on this matter before final action is taken by the Commission.

Copies of the memoranda noted above have been provided to you.

The Committee met with representatives of the NRC Staff and representatives of IDCOR to continue its discussion of the development of a Severe Accident Policy (NUREG-1070, "NRC Policy on Future Reactor Designs: Decisions on Severe Accident Issues in Nuclear Power Plant Regulation") for dealing with accidents more severe than those now analyzed as design basis accidents. Committee action on this item has been deferred for further Committee consideration during the 291st ACRS meeting.

Subcommittee Activities

Since the last report of ACRS activities, the following meetings have been held:

Fort St. Vrain, in Platteville, CO on May 17, 1984 to review plant operating experience

Metal Components, in Washington, D.C. on May 17-18, 1984 to discuss reactor coolant water chemistry and its effects on material behavior, and review the status of pressurized thermal shock and BWR pipe cracking matters and NRR/RES programs in these areas

Combined Extreme External Phenomena and Diablo Canyon, in Los Angeles, CA on May 24, 1984 to review matters related to Diablo Canyon as requested in an April 13, 1984 letter from Chairman Palladino to J. C. Ebersole

Committee Activities, in Washington, D.C. on May 30, 1984 to exchange views regarding the future scope and direction of Committee activities

Class 9 Accidents, in Washington, D.C. on May 31, 1984 to discuss the latest redraft of NUREG-1070, "NRC Policy on Future Reactor Designs: Decision on Severe Accident Issues in Nuclear Power Plant Regulation"

Combined Reactor Radiological Effects/Air Systems/Waste Management, in Washington, D.C. on May 31 and June 1, 1984 to review NRC proposed research programs in the pertinent areas for FY 1986 and 1987

Electrical Systems, in Washington, D.C. on June 1, 1984 to review the work being sponsored by the RES Instrumentation and Control Branch in preparation for writing the report to the Commission on the FY 1986 and 1987 budget

Emergency Core Cooling Systems, in Washington, D.C. on June 4, 1984 to review portions of the NRC Safety Research Program for the ACRS Report to the Commission on the FY 1986 and 1987 budget

Decay Heat Removal Systems, in Washington, D.C. on June 5, 1984 to continue review of the effort to resolve Unresolved Safety Issue A-45, "Shutdown Decay Heat Removal Requirements"

River Bend Station Unit 1, in Baton Rouge, LA on June 7-8, 1984 to review the application of the Gulf States Utilities Company for an operating license

Regulatory Activities, in Washington, D.C. on June 12, 1984 to review Regulatory Guide 1.35, Revision 3, "Inservice Inspection of UngROUTED Tendons in Prestressed Concrete Containments;" Regulatory Guide 1.35.1, "Determining Prestressing Forces for Inspection of Prestressed Concrete Containments;" and proposed general revisions to Appendix J to 10 CFR 50, "Leak Tests for Primary and Secondary Containments of Light-Water-Cooled Nuclear Power Plants"

Safeguards and Security, in Washington, D.C. on June 12, 1984 to review a proposed rule requiring replacement of highly enriched uranium fuel with low enriched uranium at nonpower reactors and to review steps for improving security measures at nonpower reactors

Advanced Reactors, in Washington, D.C. on June 12, 1984 to review NRC activities related to LMFBR research

Safety Research Program, in Washington, D.C. on June 13, 1984 to continue review of the proposed NRC Safety Research Program and Budget for FY 1986 and 1987

Future Agenda

The following items are tentatively scheduled for consideration during the 291st ACRS meeting, July 12-14, 1984:

River Bend Nuclear Station--Operating license

Severe Accident Policy--ACRS comments on revised policy statement

Proposed NRC final policy statement regarding On-shift Engineering Expertise--ACRS comments

Status report regarding USI A-45, Decay Heat Removal Systems--ACRS review of status, scope, and direction of this activity

Characterization of Generic Safety and Licensing Issues regarding their application to standardized nuclear plants--ACRS comments

Honorable Nunzio J. Palladino

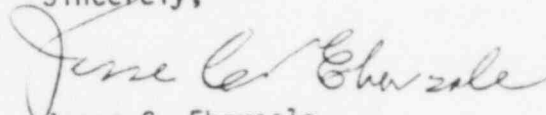
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NRC Staff briefing regarding modular review of the Westinghouse Advanced PWR standardized design

Meeting with NRC Commissioners to discuss an NTSB type board for nuclear power plants

Sincerely,

A handwritten signature in cursive script, appearing to read "Jesse C. Ebersole".

Jesse C. Ebersole
Chairman