UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

October 29, 1985

Docket No.: 50-219
LS05-85-10-037

LICENSEES: GPU Nuclear Corporation
Jersey Central Power and Light Company

FACILITY: Oyster Creek Nuclear Generating Station

SUBJECT: AUGUST 1985 PROGRESS PEVIEW MEETING ON LICENSING ACTIONS

On Wednesday, September 18, 1985, a meeting was held with GPU Nuclear (the licensee) at its headquarters in Parsippany, New Jersey to discuss the status of station licensing actions. Attachment 1 is the list of the individuals attending the meeting. Attachment 2 is a copy of the material presented by the licensee at this meeting. Reference will be made to the Cycle 11 Refueling (Cycle 11R) outage which is currently scheduled to begin in April 1986. The following is a summary of the significant items discussed and the actions taken or proposed.

The following abbreviations are referred to in the summary given below: Appendix A Technical Specifications (TSs), the staff's Oyster Creek Project Manager (OCPM), Generic Letter (GL) and heating, ventilation and air conditioning (HVAC). The TAC numbers in each heading can be found in the staff's Licensing Actions Report. This report was updated by the OCPM following this meeting.

1.0 Generic Letter (GL) 83-15

GL 83-15, Information Relating to the Deadlines for Compliance with 10 CFR 50.49, "Environmental Qualification of Electrical Equipment Important to Safety for Nuclear Power Plants", was issued by the staff on August 6, 1985. Its purpose was to advise licensees that it is the Commission's intention that extensions to 10 CFR 50.49(g) beyond November 30, 1985, will be granted only in rare circumstances and that enforcement will be taken against licensees that continue to operate their plants with unqualified electrical equipment beyond November 30, 1985, without approval by the Commission. The GL also stated that licensees desiring an extension beyond November 30, 1985, must submit an extension request at the earliest possible date to the Commission and requests received after September 30, 1985, will not be considered timely and may be denied on that basis.

The licensee stated that it is not considering a request to extend the date for compliance with 10 CFR 50.49 beyond November 30, 1985. This date was granted to the licensee in the staff's letter of March 30, 1985. The licensee stated that its plans are to complete environmental qualification modifications for Oyster Creek in its upcoming Cycle 10M outage in October and November 1985 and the station will be in compliance by November 30, 1985. The Cycle 10M outage began on October 18, 1985.

2.0 Control Room Habitability (TAC 46466)

The licensee presented the results of its chlorine transport analysis for the control room in its letter of August 16, 1985. In the second statement under assumptions and input data for the analysis, the licensee stated that the "Ventilation infiltration to the control room HVAC system during the isolation mode is assumed to be 450 cu. ft./min. for this analysis." The licensee stated that the reference to the "isolation mode" in this letter refers to the mode where the control room HVAC is in recirculation with a minimum air flow into the control room. An alarm would inform the control room operators to then isolate the control room from outside.

The licensee explained that the use of the term "isolation mode" in the letter was incorrect. The submittal should have stated "recirculation mode." The control room HVAC was modified following the meeting with the staff on control room habitability on March 19, 1985. For the old design, the reference to the "isolation mode" would have been correct. For the new design, the "isolation mode" means the control room is isolated and "recirculation mode" means the control room has a minimum air flow from outside.

In the meeting of March 30, 1985, at the Oyster Creek site, the licensee agreed to install a switch to (1) isolate the control room and (2) put the ventilation in recirculation mode with the control room being pressurized. The staff's meeting minutes for this meeting were issued on April 16, 1985. The licensee's letter on this meeting is dated June 4, 1985.

3.0 Masonry Walls/Deferment of Work From The Cycle 11 Refueling Outage (TAC 59400)

By letter dated July 26, 1985, the licensee requested the staff to defer eight items from being completed in the Cycle 11R outage to being completed in the operating Cycle 11 or Cycle 12R outage. One of these items is masonry walls of IE Bulletin 80-11 (TAC 42914, MPA B-59) which can be modified or fixed during operating Cycle 11 without requiring the plant to be shut down.

The OCPM requested a list of the walls to be modified in the Cycle 11R outage and in the operating Cycle 11. Attachment 2 is this list of walls and drawings showing their location at Oyster Creek. This was provided by the licensee. The OCPM presented that a staff memorandum requesting NRR branch review of these deferments was sent out on August 29, 1985.

4.0 Radiological Effluent Technical Specifications (RETS) (TAC 08100)

The licensee stated it will request a change to its RETS submittal of October 22, 1984. This change will delete the radioactive liquid discharge monitor for the liquid radwaste system from the RETS. The licensee's problems with this monitor were discussed in the meetings of July 31 and August 1, 1985, on RETS a the Oyster Creek site. The staff's meeting minutes are dated August 22, 1985.

In the meetings of July 31 and August 1, 1985, the licensee agreed to review its RETS submittal of October 22, 1984, and provide a marked-up copy of this submittal with the dates each TS can be effective when the RETS are issued. The licensee stated this will be provided by October 3, 1985.

5.0 Generic Letter 83-36, NUREG-0737 Technical Specifications (TAC 54554)

The OCPM stated that the staff is completing its evaluation of the licensee's submittals of October 5, 1984, and February 11, 1985. The staff will be requesting TSs on the post-accident high range noble gas monitor and radioiodine and particulate sampler. This equipment is installed and operable at Oyster Creek. These are NUREG-0737 Items II.F.1.1 and II.F.1.2. Acceptable TSs for these systems are the TSs attached to GL 83-36.

The use of "should" in the licensee's proposed TSs in the letter dated February 11, 1985, was discussed for the following post-accident monitors: wide range drywell pressure monitors, wide range torus water level monitors and drywell hydrogen monitors. These are, respectively, the containment pressure, water level and hydrogen monitors of NUREG-0737 Items II.F.1.4, II.F.1.5 and II.F.1.6, respectively. The licensee proposed that the monitor channels should be continuously indicated in the control room during power operation. These are TSs 3.13.D.1, 3.13.E.1 and 3.13.F.1 in the February 11, 1985, submittal. The "should" in the previous sentence is not considered by the staff to be a legally enforceable statement of what the licensee is required to do to be in compliance with the plant TSs. The licensee stated that it did not intend this and agreed to have the TSs 3.13.D.1, 3.13.E.1 and 3.13.F.1 issued by the staff with the "should" replaced by "shall" in the TSs.

6.0 Changes For Clarification and Consistency Among Accident Monitoring Instrumentation (TAC 56918)

In the February 11, 1985, submittal discussed in section 5.0 above, the licensee also proposed changes to the Relief Valve Position Indicators in Section 3.13, Accident Monitoring Instrumentation, of the TSs. These changes included the following: (1) replace "shall" by "should" in the TS 3.13.A.1 to require that the channels in Table 3.13.1 should be operable and (2) revising Table 3.13.1 to delete the references to primary and secondary Relief Valve Position Indicators.

Based on the discussion of "should" and "shall" in Section 5.0 above, the licensee agreed to withdraw request number (1) above.

The licensee proposed an emergency amendment in its letter of June 28, 1983, which revised Table 3.13.1 for the primary and secondary Relief Valve Position Indicators. This was authorized in Amendment No. 88 dated July 1, 1985. The licensee agreed that this change superseded its request of February 11, 1985, and agreed to withdraw its request in that letter to revise Table 3.13.1.

7.0 Procedures Generation Package Review (TAC 44324)

The licensee has not submitted the writer's guide for the procedures generation package of Supplement No. 1 to NUREG-0737. The licensee stated it is resolving comments from the second site review on this guide. At this time, there is no date for submitting the guide. The staff stated it will request the guide officially if necessary.

8.0 Instrumentation to Follow the Course of an Accident (RG 1.97) (TAC 51115)

In the May 14, 1985 Progress Review meeting on Licensing Actions, the licensee asked the staff to stop its review of the licensee's submittals on Regulatory Guide 1.97. The licensee stated then it was planning to submit revision 1 of its previous submittals on July 31, 1985. The licensee now has stated that revision 1 has not been completed but it is expected this will be submitted in October 1985.

9.0 Salem ATWS (Generic Letter 83-28)

The staff issued a request for information on items 2.1, 2.2, 3.1.3, 3.2.3, 4.4 and 4.5.3 of GL 83-28 on April 5, 1985. The licensee has responded with only the following submittal: June 12, 1984 for item 4.5.3. The licensee stated that its submittal to respond to the remaining items will be sent by September 30, 1985.

10.0 Exemptions to Appendix R, Fire Protection Program (TAC 56786)

The staff has completed its review of the licensee's fire protection program dated April 12, 1985, but is waiting for the licensee's letter to document its commitment made by phone call in August 1985. These commitments were made to resolve the staff's concerns given in Attachment 3. A draft of this letter has been reviewed by the staff. The licensee submitted this letter on October 9, 1985.

11.0 Cycle 12 Reload

This was the subject of a meeting held at NRR Headquarters on March 13, 1985. The staff's meeting minutes are dated March 26, 1985. The licensee is planning to prepare the reload package for Cycle 12 entirely by itself.

The licensee stated that it is preparing topical reports on (1) lattice physics and (2) three dimensional steady-state core flux for the Cycle 12 reload; however, these reports need to be reviewed and approved by the staff before the restart from the Cycle 11 refueling outage. This approval is needed for the licensee to prepare a technical specification change request for Cycle 11 operations following the restart from the Cycle 11R outage.

The OCPM stated that these topical reports should be submitted as soon as possible and the licensee should request a date for their approval which allows time to prepare the technical specification change request. The licensee did not have a date when the topical reports would be submitted to the staff but stated they would not be submitted together.

12.0 Generic Issue 77

The staff requested the licensee's participation in the staff's Generic Issue 77 in its letter of August 22, 1985. This issue concerns the flooding of safety equipment compartments by backflow through floor drains. This had been discussed with the licensee before the staff requested the licensee to participate in resolving this issue. The licensee stated that it is willing to participate in the program but it is not willing to have formal answers to the three questions in the staff's letter. The licensee stated it is willing to have a meeting at the Oyster Creek site to discuss the staff's questions. This will be stated in a letter to the staff. The licensee expected to send the letter by September 30, 1985.

13.0 Next Meeting

The September Progress Review Meeting will be held at the State of New Jersey, Bureau of Radiation Protection Offices in Trenton, New Jersey on October 24, 1985.

Jack N. Donohew, Jr., Project Manager
Operating Reactors Branch No. 5
Division of Licensing

Attachments:
1. List of Attendees
2. Licensee Handouts

cc: D. Crutchfield
J. Zwolinski

DISTRIBUTION
Docket
NRC PDR
Local PDR
ORB #5 Rdg
JZwolinski
JDonohew
CJamerson
OELD
EJordan
BGrimes
ACRS (10)
NRC Participants

| DL:ORB#5 | DL:ORB#5 | DL:ORB#5 |
|---|---|---|
| CJamerson | JDonohew:jg | JZwolinski |
| 10/28/85 | 10/29/85 | 10/29/85 |

cc:
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Mayor
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Commissioner
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Eugene Fisher, Assistant Director
Division of Environmental Quality
Department of Environmental
Protection
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Mr. P. B. Fiedler
Vice President and Director
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Station
Post Office Box 388
Forked River, New Jersey 08731

Attachment 1

MEETING
September 18, 1985

| Name | Affiliation |
|---|---|
| J. Donohew | NRC/NRR/DL |
| M. Laggart | GPUN* |
| P. Czaya | GPUN |
| G. Bond | GPUN |
| J. Lackemeyer | GPUN |
| R. Flury | GPUN |
| E. Wallace | GPUN |

*GPUN = GPU Nuclear Corporation

Attachment 2

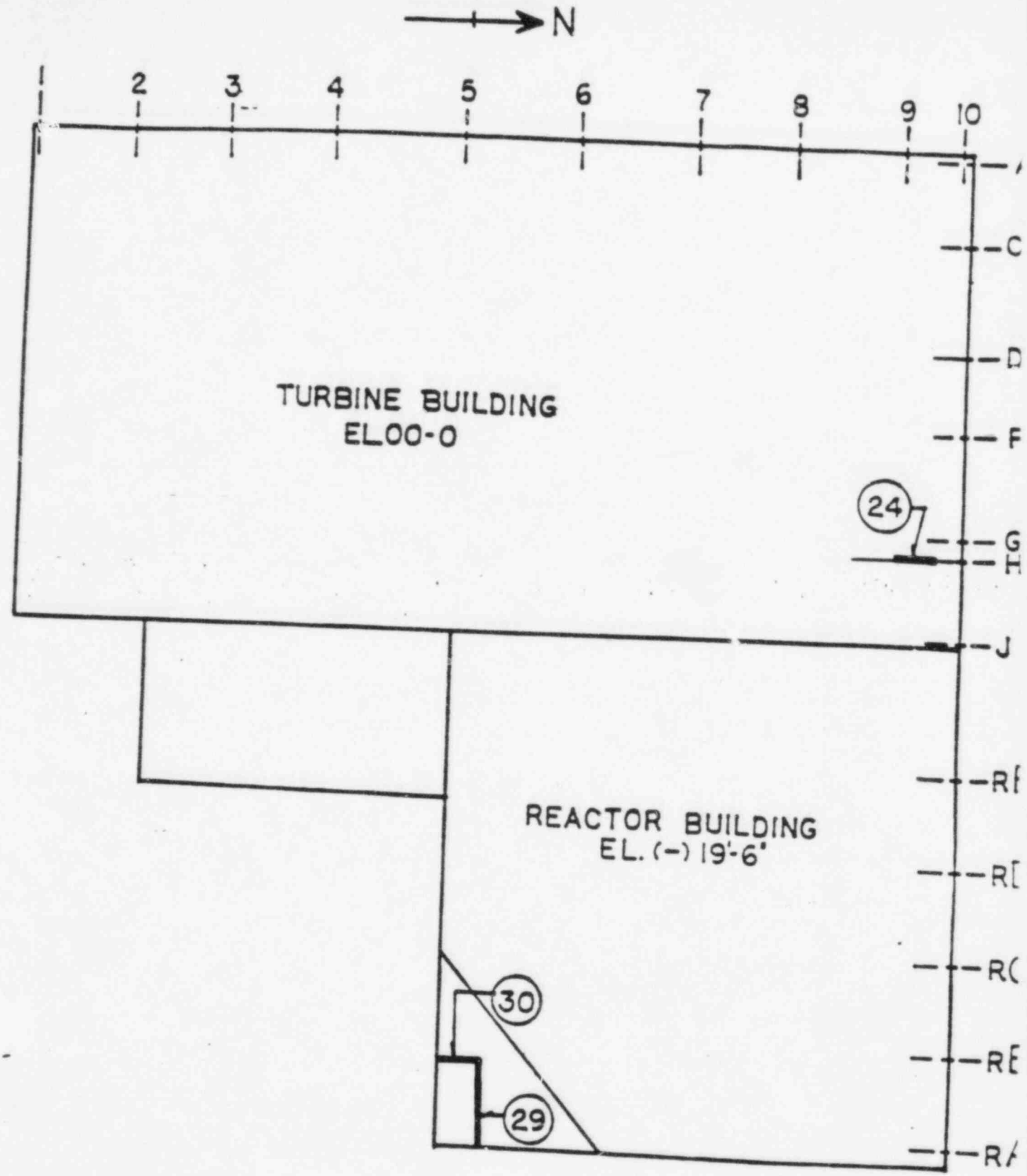
## MASONRY WALLS TO BE FIXED

### Cycle 11 Refueling

| Wall No. | Reason |
|---|---|
| 8 | Plant Computer Shutdown required. |
| 19 & 20 ~~possible~~ delay | Chemistry Lab will be partially closed during construction. |
| 17, 18, & 53 | Work inside battery room only. |
| 43 | Work on outside face only - Reactor Level Instrumentation tubing near modification. |

### Cycle 11 Operation

| Wall No. | Reason |
|---|---|
| 15 | Modification will require scaffolding etc. in monitor and change area - no effect on plant safety. |
| 24 | Modification will require scaffolding on R.B. staircase - no effect on plant safety. |
| 17, 18, & 53 | Outside face only - no effect on plant safety. |
| 29 & 30 | Mod. totally within Reactor Bldg. staircase - no effect on plant safety |
| 31, 32, 33, & 45 | Same as 29 and 30. |
| 43 | Face inside shutdown cooling Ht. Ex. Room only - minimize radiation exposure - no effect on plant safety. |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C
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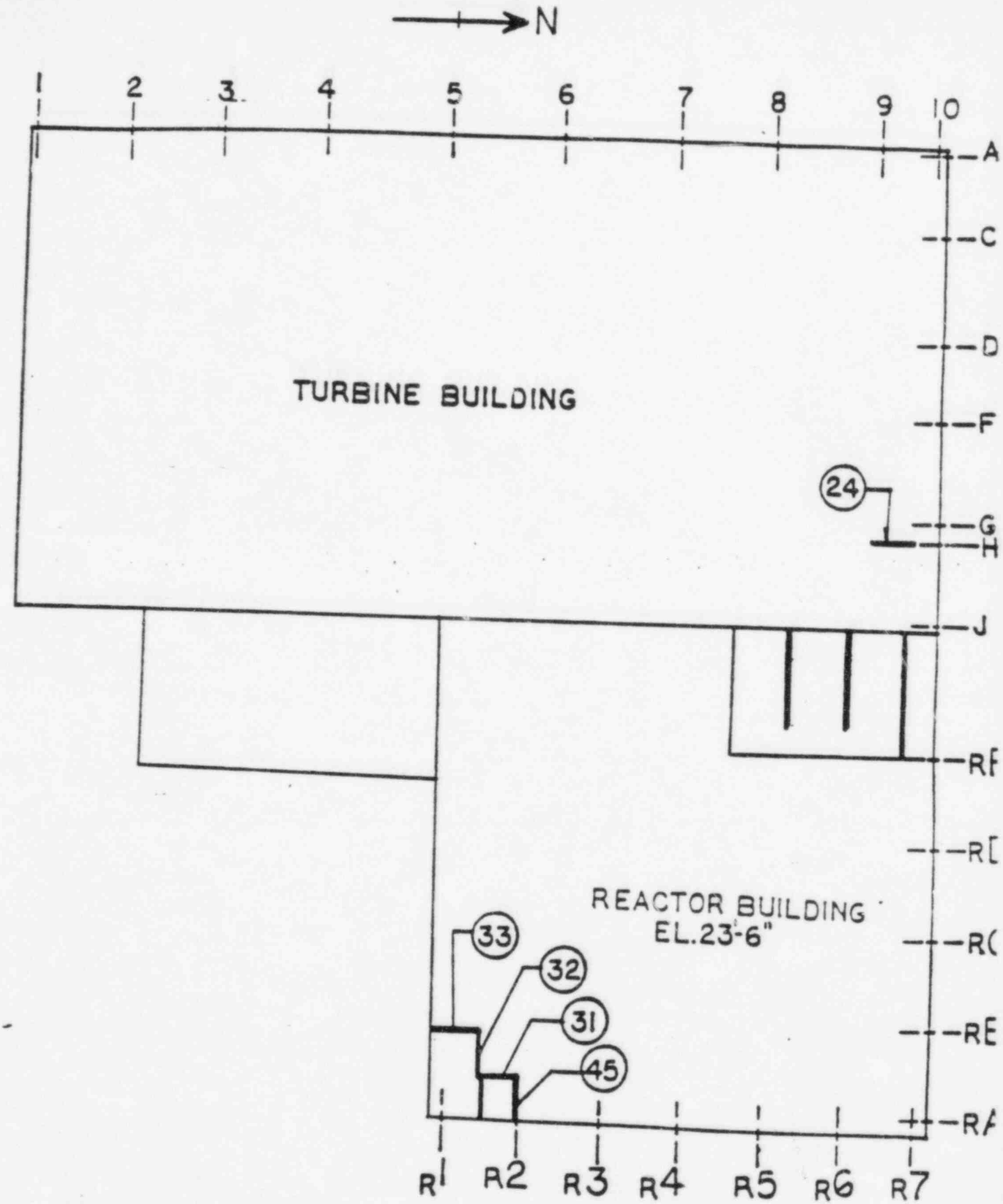
TURBINE BUILDING
EL 00-0
24
REACTOR BUILDING
EL. (-) 19'-6"
30
29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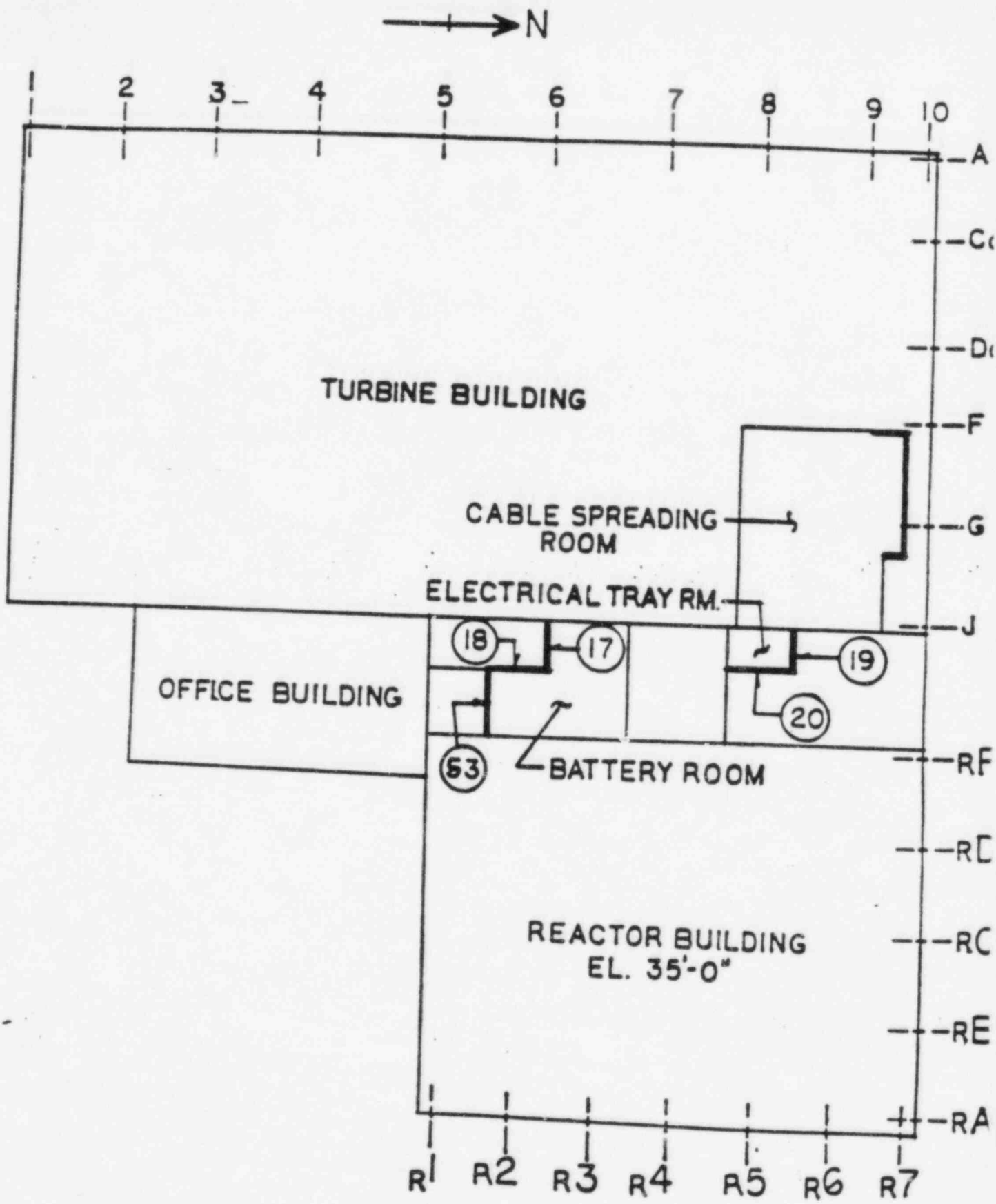
R6
R7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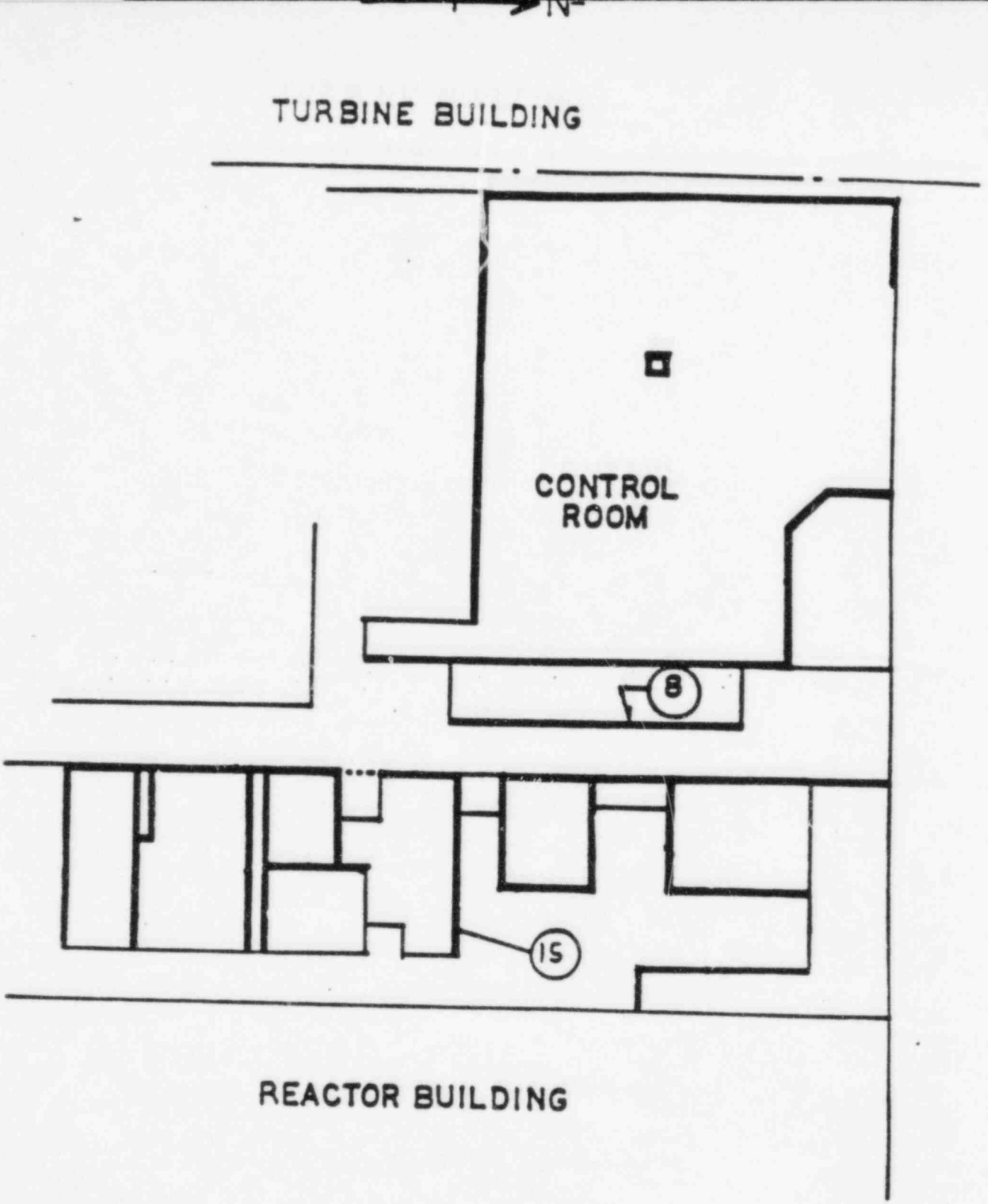
R2
R3
R4
R5
R6
R7
TURBINE BUILDING
CABLE SPREADING ROOM
ELECTRICAL TRAY RM.
OFFICE BUILDING
18
17
19
20
53
BATTERY ROOM
REACTOR BUILDING
EL. 35'-0"


TURBINE BUILDING
CONTROL ROOM
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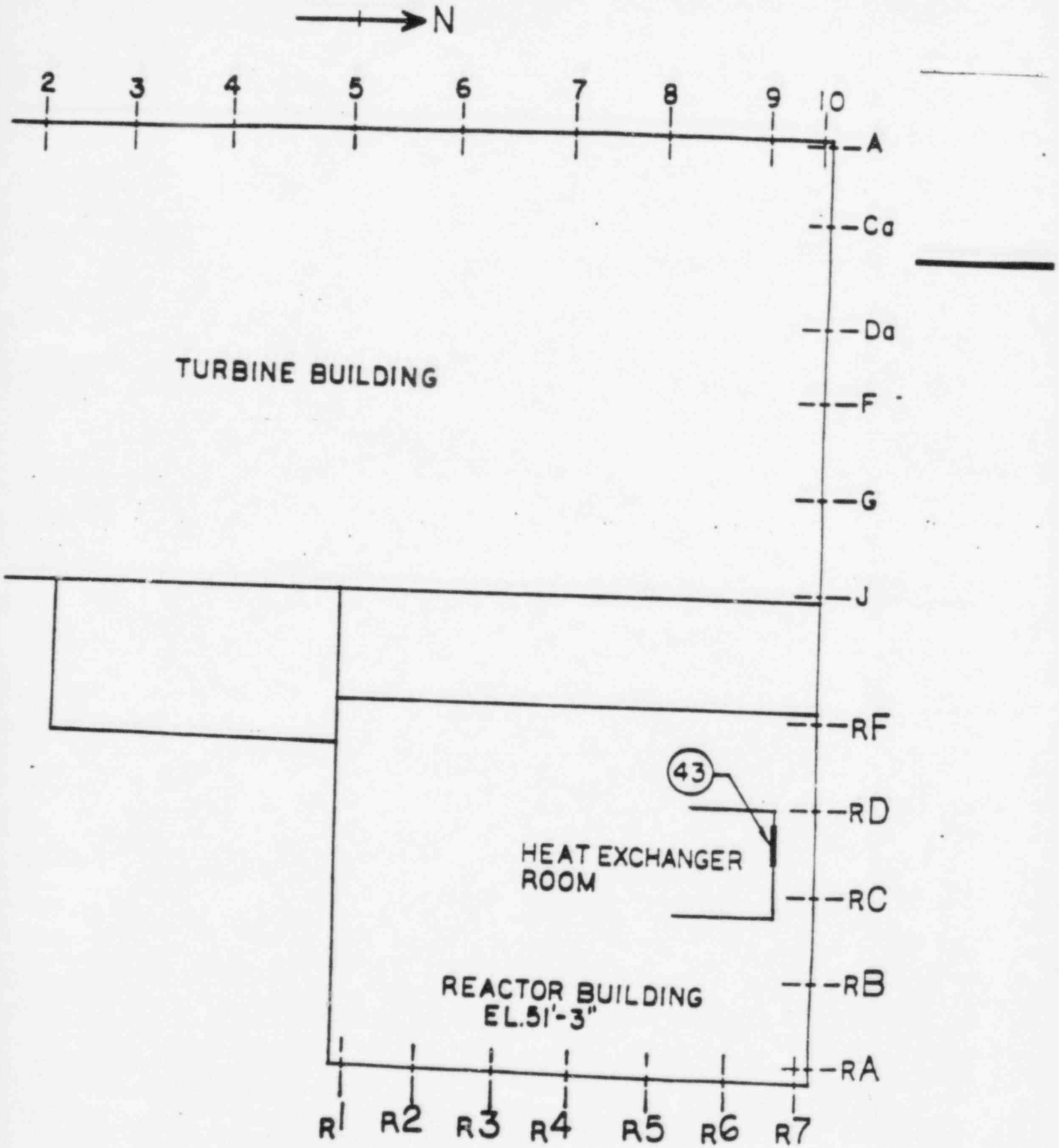
15
REACTOR BUILDING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RC
RB
RA
TURBINE BUILDING
43
HEAT EXCHANGER ROOM
REACTOR BUILDING EL.51'-3"
R1
R2
R3
R4
R5
R6
R7

Attachment 3

# STAFF'S CONCERNS WITH LICENSEE'S FIRE PROTECTION PROGRAM IN ITS SUBMITTAL DATED APRIL 12, 1985

In numerous sections of the submittal, the statement is made that cables will be rerouted or protected with fire barriers. These sections should specify the option to be implemented.

The submittal identified fire detection and suppression systems provided in various fire areas; however, the information provided does not indicate where these systems are area wide or partial area protection.

The exemption requests do not specify the section of Appendix R for which exemption is being requested.

Manual actions are required within 45 minutes for postulated fires in numerous locations. Credit can not be taken for manual actions, within the affected fire area, in less than 1 hour.

Two cable pull pits are identified in the submittal for which it is proposed to seal the cover plate. In addition to sealing the cover plate fire detection capability should be provided and the seal should be surveilled in accordance with the Technical Specification for fire barriers.

The submittal does not discuss fire area boundries. Verify that fire area boundries are of rated construction.

Many of the zone boundries appear to be unacceptable for providing adequate separation of shutdown systems. Zone boundries which separate redundant shutdown systems should be continuous with fully sealed or protected openings. Boundries shold be surveilled in accordance with the applicable Technical Specifications.

In Fire Zones OB-FZ-22B and OB-FZ-22C the entry portals provide common access to both zones. Verify that in the event of a fire in one zone safe shutdown can be accomplished independant of the other zone.

The justification for the exemption requested in fire zone RB-FZ-1F refers to a deluge system in this zone which appears to be an error.

Clarify the ability to safely shutdown as described on page 1-62.

Clarify the commitment to seal penetrations as described on page 1-55.