NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

# LICENSEE EVENT REPORT (LER)

| FACILITY NAME (1) | DOCKET NUMBER (2) | PAGE (3) |
|---|---|---|
| Calvert Cliffs, Unit 2 | 0 5 0 0 0 3 1 8 | 1 OF 0 2 |

TITLE (4)
Pressurizer Safety Valve 201 Setpoint out of Specification

| EVENT DATE (5) | | | LER NUMBER (6) | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | |
|---|---|---|---|---|---|---|---|---|---|---|
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES | DOCKET NUMBER(S) |
| | | | | | | | | | NA | 0 5 0 0 0 |
| 1 0 | 1 9 | 8 5 | 8 5 | - 0 1 0 | - 0 1 | 1 2 | 1 9 | 8 5 | | 0 5 0 0 0 |

| OPERATING MODE (9) | POWER LEVEL (10) | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11) | | | |
|---|---|---|---|---|---|
| 3 | 0 1 0 0 | 20.402(b) | 20.405(c) | 50.73(a)(2)(iv) | 73.71(b) |
| | | 20.405(a)(1)(i) | 50.36(c)(1) | 50.73(a)(2)(v) | 73.71(c) |
| | | 20.405(a)(1)(ii) | 50.36(c)(2) | 50.73(a)(2)(vii) | OTHER (Specify in Abstract below and in Text, NRC Form 366A) |
| | | 20.405(a)(1)(iii) | 50.73(a)(2)(i) | 50.73(a)(2)(viii)(A) | |
| | | 20.405(a)(1)(iv) | X 50.73(a)(2)(ii) | 50.73(a)(2)(viii)(B) | |
| | | 20.405(a)(1)(v) | 50.73(a)(2)(iii) | 50.73(a)(2)(x) | |

LICENSEE CONTACT FOR THIS LER (12)

| NAME | TELEPHONE NUMBER |
|---|---|
| | AREA CODE |
| P. M. Knoetgen - Engineer, PMD | 3 0 1 2 6 0 - 4 8 6 9 |

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS | | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS | |
|---|---|---|---|---|---|---|---|---|---|---|---|
| X | A B | R V | D 2 4 3 | Y | | | | | | | |
| | | | | | | | | | | | |

SUPPLEMENTAL REPORT EXPECTED (14)

| | | EXPECTED SUBMISSION DATE (15) | MONTH | DAY | YEAR |
|---|---|---|---|---|---|
| YES (If yes, complete EXPECTED SUBMISSION DATE) | X NO | | | | |

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)


On 19 October 1985, with Unit 2 in MODE 3 in preparation for a refueling outage, performance of a Surveillance Test Procedure (STP) at 1550 determined that a pressurizer safety valve (2-RC-201-RV) lift setpoint was out of specification. Technical Specification action statement 3.4.2.1 was entered. The safety valve setpoint was reset to within specification at 1700, 19 October, and after completion of testing, the cooldown for refueling was continued.

Interim corrective action included verification of the hydraulic testing unit's performance, which was found to be satisfactory. On December 1, 1985, the setpoint of 2-RC-201-RV was rechecked three times and found to be within 2 psi of the as left condition of 19 October 1985.

The cause of the out of specification setpoint could not be determined.




IE22
1/1

| FACILITY NAME (1) | DOCKET NUMBER (2) | LER NUMBER (6) | | | PAGE (3) |
|---|---|---|---|---|---|
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | |
| Calvert Cliffs, Unit 2 | 0 5 0 0 0 3 1 8 | 8 5 | — 0 1 0 | — 0 1 | 0 2 OF 0 2 |

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 19 October 1985, with Unit 2 reactor in MODE 3 in preparation for a refueling outage, Surveillance Test Procedure (STP) M-2-2, "Pressurizer (PZR) Safety Valves (RV)" was performed to check, and if required, adjust the pressurizer safety valve lift setpoints. The two pressurizer safety valves are Dresser Industries type 31700 Maxiflow Safety valves. At 1550, 19 October, with reactor coolant system (AB) temperature at 500°F, one safety valve (2-RC-201-RV) was determined to be out of specification, and Technical Specification action statement 3.4.2.1 was entered. The other safety valve (2-RC-200-RV) was tested satisfactorily. The required lift setpoints and their as found condition are listed below:

| VALVE | REQUIRED SETPOINT | AS FOUND SETPOINT | AS LEFT SETPOINT |
|---|---|---|---|
| 2-RC-200-RV | 2500 (± 1%) psia | 2485 psia | 2485 psia |
| 2-RC-201-RV | 2565 (± 1%) psia | 2619 psia | 2578 psia |

The immediate corrective action was to reset the 2-RC-201-RV setpoint at 1700, 19 October. The as left setpoint of 2578 psia was based on two tests of the setpoint pressure after adjustments to the valve had been made.

Additionally, the hydroset (hydraulic testing unit) used to perform the STP was tested by the vendor to ensure proper operation. The results of the testing showed that the hydroset's performance was within the manufacturer's required tolerance.

On 1 December 1985, with Unit 2 in **MODE** 3 following refueling, STP M-2-2 was again performed on 2-RC-201-RV to verify the previous STP data. Three tests of the setpoint with the hydroset each yielded setpoints of 2576 psia, which was consistent with the 19 October 1985 as left setpoint of 2578 psia.

The safety consequences of this event were evaluated by the NSSS vendor, Combustion Engineering. Peak reactor coolant system (AB) pressure is not expected to exceed the upset limit for loss of feedwater, feedwater line break, or loss of load. The radiation release consequences of the original analyses were not affected.

The cause of the out of specification setpoint of 2-RC-201-RV could not be determined.

There have been no similar events on Unit 2 and one similar event (Ler 80-60) on Unit 1. Safety valve 2-RC-201-RV was last tested in October, 1982 and was within specification.

The contact for further discussion is Peter M. Knoetgen, (301) 260-4869.

BALTIMORE GAS AND ELECTRIC COMPANY

P.O. BOX 1475

BALTIMORE, MARYLAND 21203

NUCLEAR POWER DEPARTMENT
CALVERT CLIFFS NUCLEAR POWER PLANT
LUSBY, MARYLAND 20657

December 19, 1985

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Docket No. 50-318
License No. DPR 69

Dear Sirs:

The attached LER 85-10, Rev. 1 is being sent to you as required by 10 CFR 50.73.

Should you have any questions regarding this report, we would be pleased to discuss them with you.

Very truly yours,

L. B. Russell
Plant Superintendent

LBR:PMK:pah

cc: Dr. Thomas E. Murley
Director, Office of Management Information
and Program Control
Messrs: A. E. Lundvall
J. A. Tiernan
W. J. Lippold

IE22
1/1