

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50 - 277

UNIT PEACH BOTTOM UNIT 2

DATE DECEMBER 13, 1985

COMPANY PHILADELPHIA ELECTRIC COMPANY

W.M. ALDEN
ENGINEER-IN-CHARGE
LICENSING SECTION
NUCLEAR GENERATION DIVISION

TELEPHONE (215) 641-5022

MONTH NOVEMBER 1985

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	1035	17	1045
2	1040	18	1044
3	1036	19	1045
4	1039	20	1043
5	1038	21	1044
6	1035	22	1047
7	1035	23	1038
8	1037	24	1042
9	1037	25	1042
10	938	26	1043
11	1018	27	1040
12	1032	28	1041
13	1037	29	367
14	1037	30	0
15	1045		
16	1047		

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LICENSING SECTION
NUCLEAR GENERATION DIVISION

TELEPHONE (215) 841-5022

MONTH NOVEMBER 1985

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0		
16	0		

OPERATING DATA REPORT

DOCKET NO. 50 - 277

DATE DECEMBER 13, 1985

COMPLETED BY PHILADELPHIA ELECTRIC COMPANY

W.M. ALDEN

ENGINEER-IN-CHARGE

LICENSING SECTION

NUCLEAR GENERATION DIVISION

TELEPHONE (215) 841-5022

OPERATING STATUS

1. UNIT NAME: PEACH BOTTOM UNIT 2

NOTES: UNIT 2 EXPERIENCED ONE

2. REPORTING PERIOD: NOVEMBER, 1985

SCHEDULED LOAD REDUCTION,

3. LICENSED THERMAL POWER(MWT): 3293

AND ONE AUTO-SCRAM.

4. NAMEPLATE RATING (GROSS MWE): 1152

5. DESIGN ELECTRICAL RATING (NET MWE): 1065

6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1098

7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1051

8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE):

10. REASONS FOR RESTRICTIONS, IF ANY:

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	720	8,016	100,008
12. NUMBER OF HOURS REACTOR WAS CRITICAL	684.0	2,824.0	65,107.6
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
14. HOURS GENERATOR ON-LINE	684.0	2,543.2	63,099.8
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	2,229,096	7,774,848	186,194,849
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	729,220	2,481,570	61,200,230
18. NET ELECTRICAL ENERGY GENERATED (MWH)	704,599	2,324,701	58,587,039
19. UNIT SERVICE FACTOR	95.0	31.7	63.1

20. UNIT AVAILABILITY FACTOR	95.0	31.7	63.1
21. UNIT CAPACITY FACTOR (USING MDC NET)	93.1	27.6	55.7
22. UNIT CAPACITY FACTOR (USING DER NET)	91.9	27.2	55.0
23. UNIT FORCED OUTAGE RATE	0.0	20.9	12.9

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 12/18/85 .

26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):	FORECAST	ACHIEVED
INITIAL CRITICALITY	-----	09/16/73
INITIAL ELECTRICITY	-----	02/18/74
COMMERCIAL OPERATION	-----	07/05/74

OPERATING DATA REPORT

DOCKET NO. 50 - 278

DATE DECEMBER 13, 1985

COMPLETED BY PHILADELPHIA ELECTRIC COMPANY

W.M. ALDEN

ENGINEER-IN-CHARGE

LICENSING SECTION

NUCLEAR GENERATION DIVISION

TELEPHONE (215) 841-5022

OPERATING STATUS

- | | |
|--|--|
| <p>1. UNIT NAME: PEACH BOTTOM UNIT 3</p> <p>2. REPORTING PERIOD: NOVEMBER, 1985</p> <p>3. LICENSED THERMAL POWER(MWT): 3293</p> <p>4. NAMEPLATE RATING (GROSS MWE): 1152</p> <p>5. DESIGN ELECTRICAL RATING (NET MWE): 1065</p> <p>6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1098</p> <p>7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1035</p> <p>8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:</p> <p>9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE):</p> <p>10. REASONS FOR RESTRICTIONS, IF ANY:</p> | <p>NOTES: UNIT 3 CONTINUED ITS SIXTH</p> <p>REFUELING AND MAINTENANCE</p> <p>OUTAGE.</p> |
|--|--|

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	720	8,016	95,904
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0	4,055.7	68,613.2
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
14. HOURS GENERATOR ON-LINE	0.0	3,989.3	66,854.4
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	0	10,796,856	194,996,664
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0	3,486,130	63,993,670
18. NET ELECTRICAL ENERGY GENERATED (MWH)	* -5,932	3,290,416	61,399,718
19. UNIT SERVICE FACTOR	0.0	49.8	69.7

20. UNIT AVAILABILITY FACTOR	0.0	49.8	69.7
	-----	-----	-----
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.0	39.7	61.9
	-----	-----	-----
22. UNIT CAPACITY FACTOR (USING DER NET)	0.0	38.5	60.1
	-----	-----	-----
23. UNIT FORCED OUTAGE RATE	0.0	0.8	7.1
	-----	-----	-----

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

SCHEDULED SHUTDOWN FOR REFUELING AND MAINTENANCE OUTAGE,
STARTED 7/14/85

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 01/04/86

26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION): FORECAST ACHIEVED

INITIAL CRITICALITY	-----	08/07/74
INITIAL ELECTRICITY	-----	09/01/74
COMMERCIAL OPERATION	-----	12/23/74

* - NEGATIVE VALUE REPORTED FOR CONSISTENCY WITH FEDERAL ENERGY REGULATORY COMMISSION REPORTS.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50 - 277

UNIT NAME PEACH BOTTOM UNIT 2

DATE DECEMBER 13, 1985

REPORT MONTH NOVEMBER, 1985

COMPLETED BY PHILADELPHIA ELECTRIC COMPANY

W.M. ALDEN

ENGINEER-IN-CHARGE

LICENSING SECTION

NUCLEAR GENERATION DIVISION

TELEPHONE (215) 841-5022

					METHOD OF	LICENSEE	SYSTEM	COMPONENT	CAUSE AND CORRECTIVE
		TYPE	DURATION	REASON	SHUTTING DOWN	EVENT	CODE	CODE	ACTION TO
NO.	DATE	(1)	(HOURS)	(2)	REACTOR (3)	REPORT #	(4)	(5)	PREVENT RECURRENCE
23	851110	F	000.0	F	4	N/A	XX	XXXXXX	LOAD REDUCTION TO 770 MWE DUE TO CABLE TRAY FIRE, IN RAD WASTE AREA.
24	851129	S	036.0	D	4	2-85-25	XX	XXXXXX	WHEN THE UNIT WAS AT 31% POWER DURING THE MANUAL SHUTDOWN FOR ENVIRONMENTAL QUALIFICATIONS MODIFICATIONS, TROUBLESHOOTING OF THE MAIN TURBINE STOP VALVES CAUSED AN AUTOMATIC SCRAM. THE #2 TURBINE STOP VALVE FAILED TO CLOSE DURING SURVEILLANCE TESTING. WHILE ATTEMPTING TO CLOSE THE #2 TURBINE STOP VALVE, THE #1, #3, AND #4 SLAVE TURBINE STOP STOP VALVES ALSO CLOSED CAUSING THE AUTOMATIC SCRAM.
			----- 36.0						

(1)

(2)

(3)

(4)

F - FORCED
S - SCHEDULED

REASON

A - EQUIPMENT FAILURE (EXPLAIN)

B - MAINTENANCE OR TEST

C - REFUELING

D - REGULATORY RESTRICTION

E - OPERATOR TRAINING + LICENSE EXAMINATION

F - ADMINISTRATIVE

G - OPERATIONAL ERROR (EXPLAIN)

METHOD

1 - MANUAL

2 - MANUAL SCRAM.

3 - AUTOMATIC SCRAM.

4 - OTHER (EXPLAIN)

EXHIBIT G - INSTRUCTIONS FOR PREPARATION OF DATA ENTRY SHEETS FOR LICENSEE EVENT REPORT (LER) FILE (NUREG-0163)

(5)

H - OTHER (EXPLAIN)

EXHIBIT I - SAME SOURCE

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50 - 278

UNIT NAME PEACH BOTTOM UNIT 3

DATE DECEMBER 13, 1985

REPORT MONTH NOVEMBER, 1985

COMPLETED BY PHILADELPHIA ELECTRIC COMPANY

W.M. ALDEN
ENGINEER-IN-CHARGE
LICENSING SECTION
NUCLEAR GENERATION DIVISION
TELEPHONE (215) 841-5022

NO.	DATE	(1)	(HOURS)	(2)	REACTOR (3)	METHOD OF SHUTTING DOWN	LICENSEE EVENT	REPORT #	(4)	(5)	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
13	851101	S	720.0	C	4		N/A		RC	FUELXX	SHUTDOWN FOR SIXTH REFUELING/MAINTENANCE OUTAGE

			720.0								

(1)

(2)

(3)

(4)

F - FORCED
S - SCHEDULED

REASON
A - EQUIPMENT FAILURE (EXPLAIN)
B - MAINTENANCE OR TEST
C - REFUELING
D - REGULATORY RESTRICTION
E - OPERATOR TRAINING + LICENSE EXAMINATION
F - ADMINISTRATIVE
G - OPERATIONAL ERROR (EXPLAIN)
H - OTHER (EXPLAIN)

METHOD
1 - MANUAL
2 - MANUAL SCRAM.
3 - AUTOMATIC SCRAM.
4 - OTHER (EXPLAIN)

EXHIBIT G - INSTRUCTIONS
FOR PREPARATION OF DATA
ENTRY SHEETS FOR LICENSEE
EVENT REPORT (LER)
FILE (NUREG-0161)

(5)

EXHIBIT I - SAME SOURCE

Peach Bottom Atomic Power Station
Narrative Summary of Operating Experiences
November, 1985

UNIT 2

Unit 2 began the report period operating at full power and maintained full power operation until November 10.

On November 10, a fire occurred in a cable tray in the radwaste building. An electrical transient (caused by the fire) caused a recirculation pump runback and a turbine electrohydraulic control system (EHC) runback, resulting in a load reduction to 770 MWe. On November 11, the unit returned to full power and maintained full power operation until November 29. Repairs to damaged cables were completed within a week. Investigation into the cause of the cable tray fire continues.

On November 29, the unit was removed from service for a modification and maintenance outage to complete environmental qualification (EQ) work in accordance with NRC regulations and to perform pipe support snubber inspections.

During the manual shutdown for EQ work on November 29, the reactor scrammed due to the closure of the 1, 3 and 4 turbine stop valves while attempting to close the 2 turbine stop valve. The 2 turbine stop valve had failed to close earlier the same day while being tested and was being investigated at the time of the scram.

UNIT 3

Unit 3 continued its sixth refuel outage throughout the report period.

On November 2, the 3C Residual Heat Removal (RHR) pump motor failed due to a fire in the lower motor guide bearing reservoir. Guide bearing failure was caused by a pump impeller wear ring failure. A new pump and a motor have been installed, tested, and placed into service. An investigation concludes that IGSCC of the pump impeller wear ring was the cause of failure.

Weld overlay of the RHR and Recirculation System welds was completed on November 12.

On November 16, the 3A residual heat removal (RHR) pump was disassembled to replace the pump flange gasket. During disassembly, a cracked impeller wear ring was also observed. As a result, the motor lower guide bearing was inspected, revealing signs of some damage. The pump impeller was replaced and the motor lower guide bearing was repaired. The 3A motor and pump were returned to service on November 26. Because of the problems

with the 3A and 3C RHR pumps, the 3B and 3D were also inspected and showed significant impeller wear ring damage. The 3B and 3D pumps are being repaired.

On November 23, during installation of the steam separator, four of the 48 hold-down bolts broke. Analysis of the bolt concludes IGSCC to be the cause of failure. During removal, one of the bolts was dropped into the annulus region of the vessel, resulting in damage to a jet pump pressure sensing line inside the vessel. Resolution of this problem is being investigated.

LLM

REFUELING INFORMATION

1. Name of facility:

Peach Bottom Unit 2

2. Scheduled date for next refueling shutdown:

January 31, 1987

3. Scheduled date for restart following refueling:

May 1, 1987

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

Yes

If answer is yes, what, in general, will these be?

Technical Specifications to accommodate reload fuel.
Modifications to reactor core operating limits.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

Reload 7 license amendment to be submitted December 12, 1986.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

None expected.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool:

- (a) Core - 764 Fuel Assemblies
(b) Fuel Pool - 1462 Fuel Assemblies, 58 Fuel Rods

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The spent fuel pool storage capacity has been relicensed for 2816 fuel assemblies. A request for approval of replacement higher density fuel racks was submitted to the NRC on June 13, 1985 which will increase the fuel pool capacity to a total of 3819 fuel bundles in each fuel pool.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

March, 1991 (September, 1986, with reserve full core discharge)

REFUELING INFORMATION

1. Name of facility:

Peach Bottom Unit 3

2. Scheduled date for next refueling shutdown:

July 14, 1985

3. Scheduled date for restart following refueling:

January 4, 1986

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

Yes.

If answer is yes, what, in general, will these be?

Technical Specifications to accommodate reload fuel.
Modifications to reactor core operating limits.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

MAPLHGR Limits Submitted January 7, 1985
Additional Core Limit Information - April 11, 1985

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

None expected.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool:

(a) Core - 764 Fuel Assemblies
(b) Fuel Pool - 1212 Fuel Assemblies, 6 Fuel Rods

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The spent fuel pool storage capacity has been relicensed for 2816 fuel assemblies. A request for approval of replacement higher density fuel racks was submitted to the NRC on June 13, 1985 which will increase the fuel pool capacity to a total of 3819 fuel bundles in each fuel pool.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

March, 1992 (March, 1987 with reserve for full core discharge)

PHILADELPHIA ELECTRIC COMPANY
2301 MARKET STREET
P.O. BOX 8699
PHILADELPHIA, PA. 19101
(215) 841-4000

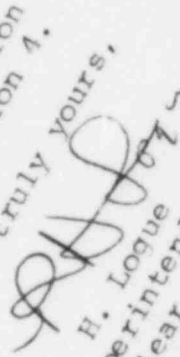
December 13, 1985
Docket Nos. 50-277
50-278

Director
Office of Inspection & Enforcement
US Nuclear Regulatory Commission
Washington, DC 20555
Attention: Document Control Desk

SUBJECT: Peach Bottom Atomic Power Station
Monthly Operating Report

Gentlemen:

Attached are twelve copies of the monthly operating
report for Peach Bottom Units 2 and 3 for the month of November,
1985 forwarded pursuant to Technical Specification 6.9.1.d under
the guidance of Regulatory Guide 10.1, Revision 4.

Very truly yours,

R. H. Logue
Superintendent
Nuclear Services

Attachment
cc:

Dr. T. E. Murley, NRC
Mr. T. P. Johnson, Resident Inspector
Mr. Stan P. Mangi, Dept. of Envir. Resources
Mr. P. A. Ross, NRC (2 copies)
INPO Records Center, Maryland Power Plant

PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-4000

December 13, 1985

Docket Nos. 50-277
50-278

Director
Office of Inspection & Enforcement
US Nuclear Regulatory Commission
Washington, DC 20555

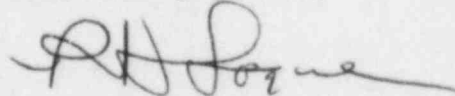
Attention: Document Control Desk

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Monthly Operating Report

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Very truly yours,



R. H. Logue
Superintendent
Nuclear Services

Attachment

cc: Dr. T. E. Murley, NRC
Mr. T. P. Johnson, Resident Inspector
Mr. Stan P. Mangi, Dept. of Envir. Resources
Mr. P. A. Ross, NRC (2 copies)
Mr. Thomas Magette, Maryland Power Plant Siting
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