

LICENSEE EVENT REPORT (LER)

| | | | | | | | | | | | | | | | | |
|---|--------|--|----------------|-------------------|-----------------|------------------|-----------------|-----------|----------------|---|--|-------------------------------|------------------|--|----------------------|------|
| FACILITY NAME (1) Millstone Nuclear Power Station Unit 1 | | | | | | | | | | DOCKET NUMBER (2) 0 5 0 0 0 2 4 5 1 | | | | | PAGE (3) 1 OF 012 | |
| TITLE (4) E.S.F. Systems Initiation and Isolation | | | | | | | | | | | | | | | | |
| EVENT DATE (5) | | | LER NUMBER (6) | | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | | | | | | |
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES | | | | DOCKET NUMBER(S) | | | |
| 09 | 27 | 85 | 85 | 0118 | 0010 | 10 | 25 | 85 | | | | | 0 5 0 0 0 0 0 0 | | | |
| OPERATING MODE (9) | | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11) | | | | | | | | | | | | | | |
| N | | 20.402(b) | | | | 20.405(a) | | | | <input checked="" type="checkbox"/> 80.73(a)(2)(iv) | | | | 73.71(b) | | |
| POWER LEVEL (10) | | 20.405(a)(1)(i) | | | | 80.36(a)(1) | | | | 80.73(a)(2)(v) | | | | 73.71(a) | | |
| 0 | | 20.405(a)(1)(ii) | | | | 80.36(a)(2) | | | | 80.73(a)(2)(vi) | | | | OTHER (Specify in Abstract below and in Text, NRC Form 306A) | | |
| | | 20.405(a)(1)(iii) | | | | 80.73(a)(2)(i) | | | | 80.73(a)(2)(vii)(A) | | | | | | |
| | | 20.405(a)(1)(iv) | | | | 80.73(a)(2)(ii) | | | | 80.73(a)(2)(vii)(B) | | | | | | |
| | | 20.405(a)(1)(v) | | | | 80.73(a)(2)(iii) | | | | 80.73(a)(2)(viii) | | | | | | |
| | | 20.405(a)(1)(vi) | | | | 80.73(a)(2)(iv) | | | | 80.73(a)(2)(ix) | | | | | | |
| LICENSEE CONTACT FOR THIS LER (12) | | | | | | | | | | | | | | | | |
| NAME David J. Yapchanyk, Engineer - Ext. 4428 | | | | | | | | | | TELEPHONE NUMBER | | | | | | |
| | | | | | | | | | | AREA CODE 210 344 71-1 7191 | | | | | | |
| COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) | | | | | | | | | | | | | | | | |
| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC | | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC | | | | | | |
| | | | | | | | | | | | | | | | | |
| | | | | | | | | | | | | | | | | |
| SUPPLEMENTAL REPORT EXPECTED (14) | | | | | | | | | | | | EXPECTED SUBMISSION DATE (15) | | MONTH | DAY | YEAR |
| <input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) | | | | | | | | | | | | NO | | 12 | 31 | 85 |

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On September 27, 1985, at 1334 hours, while in the shutdown mode (486°F, 600 psig) a loss of normal power (LNP) occurred causing various isolations and system initiations. The LNP was caused by hurricane Gloria. The reactor water cleanup system isolated, and the standby gas treatment initiated, as designed, as a result of the LNP. The isolation condenser, which was being put in service remote-manually from the Control Room, isolated on a high steam/condensate flow signal (Group 4). This isolation was not related to the LNP. Operations personnel followed the operator actions delineated in ONP 503B, "Loss of All Station AC Power". The isolation condenser was placed in service and used to maintain the reactor in hot shutdown.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

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|--|--|----------------|----------------------|--------------------|----------|----|-------|
| FACILITY NAME (1) Millstone Nuclear Power Station Unit 1 | DOCKET NUMBER (2) 0 5 0 0 0 2 4 5 8 5 | LER NUMBER (6) | | | PAGE (3) | | |
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | | | |
| | | | 0 1 8 | 0 0 | 0 2 | OF | 0 2 |

TEXT (If more space is required, use additional NRC Form 366A (9/17))

On September 27, 1985, the reactor was in the shutdown mode (486°F, 600 psig) in anticipation of a loss of normal power (LNP) resulting from hurricane Gloria. At 1330 hours while placing the isolation condenser in service, the condensate return valve (IC-3) was throttled remotely and the isolation condenser isolated on high steam/condensate flow, a Group 4 isolation. The operator who was stationed locally at IC-3 verified that no steam leaks existed outside the drywell and that drywell pressure was normal, ensuring that steam leaks did not exist in the drywell. The Group 4 isolation was reset and IC-3 was then manually throttled.

At 1334 hours a loss of normal power occurred. The reactor water clean up system isolated and the standby gas treatment initiated. The isolation condenser isolated on a Group 4 which was unrelated to the LNP. Operations personnel followed the actions delineated in the off normal procedure 503B, Loss of All Station AC Power. IC-3 was opened remotely after the operator stationed locally verified that no steam leaks existed, and with the isolation condenser in service, the reactor was brought to hot shutdown.

The reactor water clean up system isolation and the initiation of the standby gas treatment system were a result of the LNP. The initiation of the Group 4 isolation of isolation condenser has been investigated. All related instrumentation associated with the isolation condenser was checked by the Unit 1 Instrument and Control (I&C) Department and was found in specification. A special test will be performed as Unit 1 is shutting down for a refueling outage to recreate the conditions experienced during the isolation and collect further data.

Another event that included Engin Safeguards functions initiation was LER 85-016.

NORTHEAST UTILITIES



THE CONNECTICUT LIGHT AND POWER COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

General Offices • Selden Street, Berlin, Connecticut

P.O. BOX 270
HARTFORD, CONNECTICUT 06141-0270
(203) 666-6911

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MP-8310

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Reference: Facility Operating License No. DPR-21
Docket No. 50-245
Licensee Event Report 85-018-00

Gentlemen:

This letter forwards the Licensee Event Report 85-018-00 required to be submitted within thirty (30) days pursuant to the requirements of 10CFR50.73.

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

Wayne D. Romberg
Station Superintendent
Millstone Nuclear Power Station

WDR/DJY:mo

Attachment: LER 85-018-00

cc: Dr. T. E. Murley, Region I

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