

October 24, 1985

JMB-016

Docket No. 50-289

LICENSEE: GPU Nuclear Corporation
FACILITY Three Mile Island Unit No. 1
SUBJECT: MEETING WITH GPU NUCLEAR CORPORATION CONCERNING AUTOMATIC
INITIATION OF AUXILIARY FEEDWATER SYSTEM

On October 9, 1985, a meeting was held at the TMI-1 training building in Middletown, Pennsylvania. The purpose of the meeting was to conduct a detailed design review of the auxiliary feedwater system automatic initiation circuitry to be installed during the cycle six refueling outage. Participants in the meeting are indicated on the enclosed list.

The licensee was prepared with drawings and knowledgeable personnel. The day went relatively smoothly. However there are still some items for which the licensee must take positive actions. Specifically they are as follows:

1. The licensee is to send to the NRC a copy of the shift and daily checks.
2. The licensee is to send to the NRC a copy of the drawings of the control room panels which relate to EFW controls.
3. During the review, several diagrams were provided to the NRC staff. These diagrams are to be documented via a letter from the licensee. Specific diagrams include the following (a) latest EFW piping diagram and (b) drawings for MSV 13 A and B.
4. The licensee is to provide a statement concerning the design philosophy for testing the EFW and its consistency with that for testing the RPS system (i.e a normal 2/4 logic becomes a 2/3 logic during testing).
5. The licensee is to provide documentation identifying the Foxboro isolation devices. If they have not been previously reviewed by the NRC staff, sufficient documentation must be submitted to show that these components are qualified isolation devices. The documentation should include data to verify that the maximum credible fault (120 V AC, 20 amps) was applied to the output of the device in the transverse made (between signal and return) and other faults were considered (i.e., open and short circuits).
6. For valves operated more often than once per year (i.e. possibly EF-V10), the licensee is to provide bypass indication or justification for not providing bypass indication. In particular for EF-V10 valves, will their cycling for testing purposes end when all the new block valves (EF-V52) are installed and operational?

7. The licensee is to describe how all components are tested at power. In particular, testing of relays 62/EFA and B, 63/EFA and B, and 95 EFA and B was questioned but this issue applies to all components. Testing at power should be done for all components. However, for any component not tested at power, the licensee is to provide the testing frequency and justification for not testing at power.
8. The licensee is to provide functional (foxboro) diagrams identified during the review. The licensee indicated that these diagrams may be provided in about two weeks on an optimistic schedule.
9. If the licensee decides to manually disable EFW auto start on low OTSG water level when power levels are below 15% due to level oscillations in the steam generator, the licensee is to commit to installing automatic removal of this operating bypass when power increases above 15%.
10. The EFW turbine section nozzle hand valves are to locked open. How often will they be exercised?
11. The staff provided a general comment concerning technical specifications. Technical specifications should be implemented at start up from the Cycle 6 refueling. Since the system is being designed now, the system should be built so that not only the manufacturer's recommended tests can be performed but also so that recommended testing in accordance with the guidelines of the standard technical specifications can be performed.

The staff needs to obtain the information discussed above as soon as possible. The licensee committed to supply as much information as possible within roughly two weeks. A delay beyond the end of October will have a significant impact on completing this review.

Original signed by

John Thoma, Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosure:
As Stated

cc w/enclosure:
See next page

*See previous white for concurrences.

ORB#4:DL
OThompson*
10/21/85

ORB#4:DL
JThoma, Jr
10/23/85

JCSB
FBurrows*
10/22/85

JCSB
JJoyce*
10/22/85

ORB#4:DL
JStolz
10/22/85

7. The licensee is to describe how all components are tested at power. In particular, testing of relays 62 EFA and B, 63x EF A and B, and 95 EF A and B was questioned but this issue applies to all components. Ideally, testing at power should be done for all components. However, for any component not tested at power, the licensee is to provide the testing frequency and reason for not testing at power.
8. The licensee is to provide functional diagrams identified during the review. The licensee indicated that these diagrams may be provided in about two weeks on an optimistic schedule.
9. If the licensee decides to disable EFW auto start on low OTSG water level when power levels are below 15% due to level oscillations in the steam generator, the licensee is to commit to installing automatic enabling of the system when power increase above 15%.
10. The EFW turbine suction nozzle hand valves are to locked open. How often will they be exercised?
11. The staff provided a general comment concerning technical specifications. Technical specifications should be implemented at start up from the Cycle 6 refueling. Since the system is being designed now, the system should be built so that not only the manufacturer's recommended tests can be performed but also so that recommended testing in accordance with the guidelines of the standard technical specifications can be performed.

The staff needs to obtain the information discussed above as soon as possible. The licensee committed to supply as much information as possible within roughly two weeks. A delay beyond the end of October will have a significant impact on completing this review.

John Thoma, Project Manager
Operating Reactors Branch #4
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10/21/85

ICSB
FBurns
10/22/85

ICSB
JJoyce
10/22/85

ORB#4:DL
JStolz
10/ /85

Enclosure

ATTENDANCE LIST FOR GPU MEETING

HELD ON OCTOBER 9, 1985

<u>NAME</u>	<u>ORGANIZATION</u>
John Thoma	NRC
Owen Thompson	NRC
Joe Joyce	NRC
Fred Burrows	NRC
Skip Young	NRC
Rick McGoey	GPUN
Roy Harding	GPUN
Robert Wolf	GPUN
George Braulke	GPUN
William Drendall	GPUN
Steve Ku	GPUN
Jerry Sadauskas	GPUN
Pete Kelly	IMPELL

MEETING SUMMARY DISTRIBUTION

Licensee: GPU Nuclear Corporation

*Copies also sent to those people on service (cc) list for subject plant(s).

Docket File

NRC PDR

L PDR

ORB#4 Rdg

Project Manager - JThoma

JStolz

BGrimes (Emerg. Preparedness only)

OELD

EJordan, IE

ACRS-10

PMorriette

NRC Meeting Participants:

O. Thompson

J. Joyce

F. Burrows

S. Young