

APPENDIX

U. S. NUCLEAR REGULATORY COMMISSION  
REGION IV

NRC Inspection Report: 50-458/85-63

License: NPF-40

Dockets: 50-458

Licensee: Gulf States Utilities  
P.O. Box 2951  
Beaumont, Texas 77704

Facility Name: River Bend Station

Inspection At: St. Francisville, Louisiana

Inspection Conducted: September 23-27, 1985

Inspectors: *M. E. Skow*  
M. E. Skow, Project Engineer, Project Section A  
Reactor Projects Branch (pars. 4 and 5)

10/15/85  
Date

*C. C. Harbuck*  
C. C. Harbuck, Project Engineer, Project Section A  
Reactor Projects Branch (pars. 2 and 3)

10/24/85  
Date

Approved: *J. P. Jaudon*  
J. P. Jaudon, Chief, Project Section A  
Reactor Projects Branch

10/24/85  
Date

Inspection Summary

Inspection Conducted September 23-27, 1985 (Report 50-458/85-63)

Areas Inspected: Routine, unannounced inspection of followup of various open items and License Conditions. The inspection involved 72 inspector-hours onsite by two NRC inspectors.

Results: Within the areas inspected, no violations or deviations were identified.

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DETAILS

1. Persons Contacted

\*W. J. Cahill, Senior Vice President  
\*R. B. Stafford, Director Quality Services  
\*J. E. Spivey, Operations QA Engineer  
\*B. E. Hey, Licensing  
\*G. R. Kimmell, Supervisor, Operations QA  
\*T. C. Crouse, QA Manager  
\*D. R. Gipson, Assistant Plant Manager  
\*P. E. Freehill, Superintendent  
\*M. W. Henkel, Engineer, Nuclear Licensing  
M. Walton, Startup Engineer  
C. Greene, Project Engineer  
R. Pertuit, HVAC, Test Engineer  
R. King, Licensing Engineer  
S. Radebaugh, Assistant Superintendent of Startup  
J. P. Schippert, Project Engineer

\*Denotes those present during exit interview.

2. Licensee Actions on Previously Identified Items

- a. (Closed) Open Item (8520-03), Control Building Chilled Water Cooler 'B' Load Sequence Relays. The NRC inspector verified that the required Category I relays had been installed and that the appropriate generic tests had been performed to ensure that the relay time settings were within the limits specified by the design documents (Set Point Data Sheets). This item is closed.
- b. (Closed) Violation (8531-01), Integrated Leak Rate Test - Valve Lineups. The NRC inspector verified that the problems identified in the valve lineup had been corrected prior to test performance. The NRC inspector also reviewed the licensee's official response to this violation and verified that the actions stated therein were as stated. This item is closed.
- c. (Closed) Open Item (8531-02), Leak Test of Containment Isolation Valves 1DFR\*MOV146 and 1SSR\*SOV139. The NRC inspector verified that these valves had been properly leak tested by licensee procedure 1-SST-57 on September 19, 1985. This item is closed.
- d. (Closed) Open Item (8548-01), Preoperational Test Results Review by the Licensee. This item is closed based on the comments under violation (8549-01) discussed below.

- e. (Closed) Violation (8549-01), Inadequate Licensee Review of Preoperational Test Results. The NRC inspector reviewed the licensee's official response to this violation. Based on this review and favorable reviews by NRC inspectors of other preoperational tests, subsequent to the initiation of the licensee's corrective actions, it appears that the licensee has corrected the problems noted in the quality of results reviews. This item is closed.
- f. (Closed) Open Item (8551-08), Open Test Exception No. 21 for the Remote Shutdown System Preoperational Test. The actual installations of the interlock between the B loop Residual Heat Removal (RHR) system injection and isolation valves is not planned for completion until the first refueling outage. However, the NRC inspector determined that the licensee's action of placing administrative controls on the operation of these valves (an operator's aid tag at the remote shutdown panel) appears to be an adequate substitute for the interlock. This item is closed.
- g. (Closed) Open Item (8551-09), Design Modification Package No. 4 (DMP-4). The NRC inspector verified that the temperature monitors and level detectors installed by DMP-4 appeared to have been tested satisfactorily by reviewing the official field copy of the completed test procedure, 1-SST-53. All acceptance criteria were met. This item is closed.
- h. (Closed) Open Item (8552-01), Abnormal Operating and Emergency Operating Procedure Problem. The NRC inspector reviewed the actions taken by the licensee in response to five concerns listed under this item.
  - (1) The NRC inspector found that the problems noted with EOP-0001 being in disagreement with Technical Specifications set points had been corrected in Revision 3 of EOP-0001. A Temporary Change Notice (TCN) was issued to correct AOP-0035 for the suppression pool temperature at which emergency depressurization of the reactor is required in the event of a stuck open safety/relief valve (110°F to 105°F), and the EOPs and AOPs were found to now reflect the correct value of 2 psig for Drywell High Pressure.
  - (2) In EOP-0001, EOP-0002, and AOP-0021 there is a list of six conditions, each of which is sufficient to require emergency RPV depressurization. Two conditions, listed separately in EOP-0002, were not included in this list. The NRC inspector verified that the list had been revised to include all eight conditions.

- (3) The scale on the fuel zone level instrumentation was modified so that its zero reference point is the same as all other RPV level instruments. The procedures did not reflect this modification. The NRC inspector verified that the EOPs and AOPs had been revised to correct this problem.
- (4) Procedure AOP-0031, "Shutdown from Outside the Main Control Room," The NRC inspector found that this procedure had not yet been issued. However, because this item is included as a 5% license condition under the area of fire protection, it is deleted as a concern under this open item.
- (5) The NRC inspector determined that the writing of Section 3.4 of EOP-0002, regarding emergency containment venting, was being followed by NRR and was listed as item #16 in the license. Therefore, it is deleted as a concern under this open item. Based on the preceding discussion, this item is closed. However, until regional staff review has been completed, the revision to Section 3.4 of EOP-0002 will remain as an open item. (8563-01)

### 3. Startup Test Procedure Review

The purpose of this area of the inspection was to ascertain whether the procedures reviewed were prepared in accordance with regulatory guidance, licensee FSAR commitments, and the licensee's startup manual.

The NRC inspector reviewed the following licensee startup test procedures:

- 1-ST-4, "Shutdown Margin," Revision 0, 9/24/85
- 1-ST-6, "SRM Performance," Revision 1, 7/26/85
- 1-ST-14, "Reactor Core Isolation Cooling," Revision 0, 2/1/85

The NRC inspector found that these procedures appeared to meet FSAR test commitments and regulatory guidance. However, in the case of 1-ST-14, it was determined that a procedure revision was planned. The present procedure apparently needed several changes before it could be used to adequately run the test. Revision 1 to 1-ST-14 will remain as an open item until regional staff review has been completed. (8563-02)

### 4. TMI Action Plan Requirements

The purpose of this inspection was to verify licensee implementation of NUREG 0737 (TMI) items. This included procedures, testing, records, and modification verification onsite. Also included was verification or followup by regional inspectors, in accordance with NUREG 0989 and its supplements. The following items were reviewed:

- a. NUREG 0737, Item I.C.1, "Short Term Accident Analysis and Procedure Revision." NUREG 0989, "The River Bend Station Safety Evaluation Report (SER)," Supplement 3, identified changes in procedures the NRC regional staff would confirm. The NRC inspector found that those errors had indeed been corrected in the procedures, as the licensee committed in their letter dated August 26, 1985.

The SER stated the staff was continuing to evaluate five specific areas, the licensee stated that these areas have been addressed in their letter dated September 10, 1985.

Notwithstanding continued staff review, regional action on this item is closed.

- b. NUREG 0737, Item I.C.8., "Pilot Monitoring of Selected Emergency Procedures for OL Applicants." The SER considered this item resolved. Specific inspection in this area was not required by the SER. This item is closed.
- c. NUREG 0737, Item II.E.1.1., "Auxiliary Feedwater System Reliability." This item is applicable only to pressurized water reactors (PWRs). This item is closed.
- d. NUREG 0737, Item II.E.3.1, "Emergency Power for Pressurizer Heaters." This item is applicable only to PWRs. This item is closed.
- e. NUREG 0737, Item II.E.4.2, "Containment Isolation Dependability." The SER required the "applicant to seal closed those isolation valves on nonessential lines that are not provided with automatic isolation signals during operation modes 1 through 3. The valves in question are those on the following penetrations: 1KJB\*231, 33; 601E, F and 605E, F." The valves associated with penetrations 601E, F and 605E, F have been redesignated essential by the licensee in amendment eleven of the FSAR. An evaluation of this redesignation will be included in a supplement to the SER.

There were three valves associated with penetrations 1KJB\*231,33. The NRC inspector found these valves had not been sealed. The licensee, in turn, initiated actions to install a locking device on the door on the control panel CPP102 on the 170' level of the auxiliary building over the control switches. The licensee stated the locking device will be covered by Administrative Procedure 22, "Conduct of Operations." In addition, the licensee also issued a TCN to permanently change Procedure SOP-0040, "Hydrogen Mixing, Purge and Recombiners," which contains the applicable valve lineup. This change locks shut the control panel door.

The NRC inspector reviewed preoperational test procedures to verify compliance with the SER and Item II.E.4.2. The inspector reviewed Test Procedure 1-AT-609-9, Revision 1, which tested the reactor



building floor drain isolation valves; 1-AT-609-1, Revision 2, which tested the reactor building equipment drain isolation valves; loop calibration report 1-ILHS.091 and .092, Revision 0, which indicate the drywell high pressure trip set points of 1.68 psig which matches the Technical Specification Table 3.3.2-2; and test 1-PT-409-3, Revision 0, which performed containment/drywell isolation valves and dampers logic test. This item is closed.

- f. NUREG 0737, Item II.K.3.18c, "ADS Actuation." SER Supplement 2 accepts RBS Technical Specification 3/4.3.3. The licensee installed the additional time delays and inhibit switches in the Automatic Depressurization System logic. The timers, annunciators, lights, inhibit switches, and logic circuits are included in drawing 851E225AA, sheet 5, Revision 14. Preoperational testing had been performed by Test Procedure 1-PT-202, Revision 2. Testing included the ADS manual inhibit switch and high drywell pressure bypass timer. Surveillance Procedures STP-051-4298, STP-051-4299, and STP-202-0201 include tests for the timers and inhibit switches. This item is closed.

5. License Conditions

License NPF 40, Attachment 1, paragraph 2, item 2.a., "Modify operating procedures to require the monitoring of cooling water for control room HVAC chillers when cooling is from the standby service water source as described in the 10 CFR 50.55(e) report identifies as DR-314." TCN 85-1710 was approved August 16, 1985, to Procedure STP-000-0001, Revision 2, "Daily Operating Logs." This change placed a minimum temperature limit to the basin water below which the stand by service water pump(s) are to be operated in the recirculation mode to maintain basin water temperature. In addition, if the recirculation mode is unable to maintain water temperature, Standard Operating Procedure SOP-66, Revision 1, specifies additional operator action as discussed in FSAR, Section 9.2.10.3. In the event basin water temperature drops below 55°F, the chillers may freeze up. SOP-66, Section 5.4, provides a procedure to manually switch from chilled water in the control building cooling coils to service water. This item is closed.

6. Exit interview

An exit interview was held on September 27, 1985, with Mr. Cahill and other personnel as denoted in paragraph 1 of this report. The NRC resident inspector also attended this meeting. At the meeting, the scope of the inspection and findings were summarized.