



Northern States Power Company

414 Nicollet Mall
Minneapolis, Minnesota 55401
Telephone (612) 330-5500

October 18, 1985

J J Harrison, Chief
Engineering Branch
U S Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, Illinois 60137

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

In response to your letter of September 20, 1985, concerning Inspection Reports No. 50-282/85015 (DRS) and 50-306/85012 (DRS), the following information is offered related to the two noncompliance items cited in the report.

Violation

10 CFR 50, Appendix B, Criterion XVI, as implemented by Northern States Power Company Corporate Nuclear Administrative Control Directives, Section 10, requires conditions adverse to quality such as deficiencies, defective materials, and deviations be corrected. In the case of a significant adverse condition, the cause of the condition shall be determined and corrective action taken to preclude repetition.

Contrary to the above, after identifying a seal leak and fluid contamination in a steam generator snubber (SGS), the licensee did not investigate the cause of seal deterioration and fluid contamination. In addition, the licensee did not perform tests on the other SGSs to assure operability and to identify the possible worst case condition.

This is a Severity level IV violation (Supplement II).

Response

A program of steam generator snubber inspection and testing is underway as described in our letters of September 16, 1985 and October 16, 1985. This program is intended to fully resolve all potential conditions adverse to quality related to these components.

All 16 of the steam generator snubbers of this design are or will be tested. Eight of the 16 are installed in Unit No. 2 and were tested during the current refueling outage. No conditions adverse to quality, such as defective materials, were found in these snubbers. Analysis of snubber seal material and hydraulic fluid is not complete, however. The eight snubber in Unit No. 1 will be tested during the spring 1986 refueling outage.

8510310137 851018
PDR ADOCK 05000282
G PDR

OCT 21 1985 10
4801

Violation

10 CFR 50.73 requires the licensee to submit a Licensee Event Report (LER) within 30 days after discovery of condition that resulted in the nuclear power plant being seriously degraded, or that resulted in the nuclear power plant being in an unanalyzed condition that significantly compromised plant safety, or in a condition that was outside the design basis of the plant.

Contrary to the above, the licensee failed to submit LERs documenting (1) the identified seal leak and fluid contamination problem, (2) that the SGS control valves were replaced to avoid possible piping overstress, and (3) that a Westinghouse calculation indicated a SGS load significantly greater than the original design value (282/85015-02A, B, and C; 306/85012-02A, B, and C).

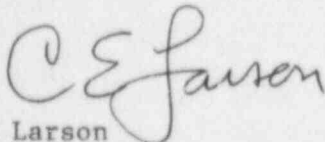
Response

Procedures are now in place which require Licensee Event Reports to be made in accordance with the requirements of 10 CFR Part 50, Section 50.73.

Westinghouse is reviewing the snubber design specifications and design load calculations. We believe these actions fully address the concern identified in the violation. A followup report will be made when this work is completed.

As noted above, portions of our investigation are continuing. This work is expected to be completed by December 31, 1985. A supplemental response will be provided to you at that time.

Please contact us if you have any questions related to the information we have provided or our schedule for providing a supplemental response.



C E Larson
Vice President Nuclear Generation

c: Regional Administrator-III, NRC
NRC Resident Inspector
G Charnoff