

LICENSEE EVENT REPORT (LER)

APPROVED ONE NO. 3180-0104
EXPIRES - 8/31/93

FACILITY NAME (1)

Limerick Generating Station - Unit 1

DOCKET NUMBER (2)

0 5 0 0 0 3 5 2

PAGE (3)

1 OF 0 3

TITLE (4)

Actuation of an Engineered Safety Feature

EVENT DATE (6)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER (8)	
0 9	1 9	8 5	8 5	0 7 4	0 0 1	0 1	0 1	8 8 5		0 5 0 0 0 1 1	
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)								
1			<input checked="" type="checkbox"/> 20.402(a) <input type="checkbox"/> 20.406(a) <input checked="" type="checkbox"/> 20.736(a)(2)(iv) <input type="checkbox"/> 20.737(a)								
POWER LEVEL (10)			<input type="checkbox"/> 20.405(a)(1)(i) <input type="checkbox"/> 20.736(a)(1) <input type="checkbox"/> 20.736(a)(2)(v) <input type="checkbox"/> 20.737(a)								
0 2 2			<input type="checkbox"/> 20.405(a)(1)(ii) <input type="checkbox"/> 20.736(a)(2) <input type="checkbox"/> 20.736(a)(2)(vi) <input type="checkbox"/> OTHER (Specify in Abstract below and in Tool, NRC Form 306A)								
			<input type="checkbox"/> 20.406(a)(1)(iii) <input type="checkbox"/> 20.736(a)(2)(i) <input type="checkbox"/> 20.736(a)(2)(vii)(A) <input type="checkbox"/>								
			<input type="checkbox"/> 20.406(a)(1)(iv) <input type="checkbox"/> 20.736(a)(2)(ii) <input type="checkbox"/> 20.736(a)(2)(viii)(A) <input type="checkbox"/>								
			<input type="checkbox"/> 20.406(a)(1)(v) <input type="checkbox"/> 20.736(a)(2)(iii) <input type="checkbox"/> 20.736(a)(2)(ix)(A) <input type="checkbox"/>								
			<input type="checkbox"/> 20.406(a)(1)(vi) <input type="checkbox"/> 20.736(a)(2)(iv) <input type="checkbox"/> 20.736(a)(2)(x)(A) <input type="checkbox"/>								

LICENSEE CONTACT FOR THE LER (12)

NAME	TELEPHONE NUMBER
John C. Nagle, Senior Engineer, Licensing Section	AREA CODE 2 1 5 8 4 1 - 5 1 8 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input checked="" type="checkbox"/>	<input type="checkbox"/>				

ABSTRACT (Limit to 1400 words, i.e., approximately fifteen single-spaced typewritten lines) (16)

Abstract: 85-074

During surveillance testing on September 19, 1985, with the unit at 22 percent power, an I&C technician inadvertently caused a fuse to blow which resulted in the automatic isolation of numerous Nuclear Steam Supply Shutoff System (NSSSS) components and initiation of Standby Gas Treatment (SBGT) system. Isolations were verified and reset per procedure and systems were returned to normal after the fuse was replaced.

Investigation into the cause of the blown fuse continues.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMS NO. 3190-0104

EXPIRES 8/31/85

FACILITY NAME (1)

Limerick Generating Station
Unit 1

DOCKET NUMBER (2)

0 5 0 0 0 3 5 2

LER NUMBER (5)

YEAR SEQUENTIAL REVISION

NUMBER NUMBER NUMBER

8 5 - 0 7 4 - 0 0

PAGE (3)

0 2 OF 0 3

TEXT (If more space is required, use additional NRC Form 266A (1))

Description of the Event:

With the Unit at 22 percent power, while performing surveillance test ST-2-42-658-1, "NSSSS-Reactor Vessel Water Level-Level 1 and 2; Division 1B, Channel B1 Functional Test (LIS-42-IN6811B, SL-42-IN684B)", the instrument and controls technician found it necessary to move cabling in order to verify the identity of a relay. The apparent movement of the cable resulted in the blowing of fuse B21H-F15A.

The de-energization of the NSSSS logic caused inboard Groups I, II, III, VI, VII and VIII isolation signals to be generated. The reactor water cleanup system, primary containment instrument gas system and secondary containment isolated as a result of that signal.

The EIIS code for this system is JM.

Consequences of the Event:

All systems performed as if valid isolation signals had been generated. The isolations were reset and the RWCU system was returned to service. There was no detrimental affect on reactor water chemistry due to the temporary loss of the RWCU system. The consequences of this event are minimal.

Cause of the Event:

In order to determine the identity of a relay, the instrument and controls technician was moving cables within panel 10C622. During a surveillance test, a verification of the state of relay was required. Coincidental with this cable movement, fuse B21H-F15A blew, generating the isolation signals.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Limerick Generating Station Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 5 2 8 5 - 0 7 4 - 0 0 0 3 OF 0 3	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 366A x (17))

Corrective Actions:

All of the isolations were confirmed and the fuse was replaced. Within a half-hour, the system was reset by procedure. The RWCU system was returned to service within 45 minutes of the isolation.

Action Taken to Prevent Recurrence:

Attempts to recreate the events which led to the blown fuse in order to further define the fault location were unsuccessful.

As a result of a previous similar occurrence (LER 85-48), several secondary fuses had been temporarily installed as an aid in determining the fault location, were the problem to recur. During this failure, none of the additional fuses had blown; this helped to narrow down the possible fault location. At this time, two more fault indicators have been installed and a physical inspection of each wire spade connector and terminal has been completed.

No abnormalities have been found. A Maintenance Report Form (MRF) has been initiated to trace each panel wire during a future outage in an attempt to locate the possible fault.

Previous Similar Occurrences:

Similar events of blown fuses (B21H-F15A) which resulted in ESF actuation were reported in LERs 85-08 and 85-48.

PHILADELPHIA ELECTRIC COMPANY

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October 18, 1985

Docket No. 50-352

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Washington, DC 20555

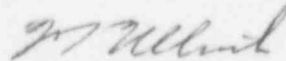
SUBJECT: Licensee Event Report
Limerick Generating Station - Unit 1

This LER concerns the spurious isolation of Nuclear Steam Supply Shutoff System (NSSSS) subsystems and the initiation of Standby Gas Treatment Systems as a result of a blown fuse.

Reference:	Docket No. 50-352
Report Number:	85-074
Revision Number:	00
Event Date:	September 19, 1985
Report Date:	October 18, 1985
Facility:	Limerick Generating Station P.O. Box A, Sanatoga, PA 19464

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv).

Very truly yours,



W. T. Ullrich
Superintendent
Nuclear Generation Division

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E. M. Kelly, Senior Resident Site Inspector
See Service List

IE22
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Atomic Safety & Licensing Board Panel
Docket & Service Section (3 Copies)
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Timothy R. S. Campbell

September, 1985