

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

FEBRUARY, 1986

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by James P. Peters, telephone number (815)357-6761 extension 325.

II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

February 1-28

Feb. 1, 0001 Hours. The Unit entered February with the reactor subcritical and unit off-line in cold shutdown for first refuel outage.

Feb. 28, 2400 Hours. Reactor in cold shutdown for first refueling outage.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED
MAINTENANCE.

1. Amendments to facility license or Technical Specification.

Amendment 33- Amendment to the CRD Low Charging Water Header
Pressure Scram Setpoint; Surveillances and LCO.

Amendment 34- Add Excess Flow Check Valve ICM102 to Table
3.6.3-1.

2. Facility or procedure changes requiring NRC approval.

There were no Facility or Procedure Changes Requiring NRC
approval during this reporting period.

3. Tests and Experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during
this reporting period.

4. Corrective maintenance of safety related equipment.

The following table (Table 1) presents a summary of
safety-related maintenance completed on Unit One during the
reporting period. The headings indicated in this summary
include: Work Request number, Component Name, Cause of
Malfunction, Results and Effects on Safe Operation, and
Corrective Action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L20783	"1A" DG Prelube Pump 1DG01K	Dry-starting of the Diesel Engine.	Prelubrication of Diesel Engine. M-1-1-83-129.	
L20787	"1A" Diesel Generator 1DG01K	Excessive vibration on floor area.	Relocate controls and monitoring instrumentation to a qualified location.	
L43784	HPCS full flow test valve to suppression pool 1E22-F023	Packing leaks external to valve	Contamination problem	Repacked valve.
L51706	Diesel Generator air relay valve 1DG061B	Internal binding within the valve.	Valve was stuck open	Rebuilt East & West Air start relay valves with new software kit.
L53312	"1B" DG Blocking diode CR2	Bad diode CR2	Failed LES-DG-102	Replaced diode CR2
L53595	Shutdown cooling testable check valve 1E12-F050A	Limit switches were misaligned.	Valve indicates open, when actually closed.	Reset limit switches
L53936	RHR service water HX outlet valve 1E12-F068B	Packing leak external to the valve.	No potential effect.	Repacked valve.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L54765	Div-II CSCS Pump room cooling fan 1VY06C	Bad bearings	No potential effect, just sounded bad.	Installed new bearings.
L55302	RHR "1B" Pump suction valve 1E12-F004B	Loose packing	Contamination problem	Adjusted packing till leak stopped.
L55400	RHR "1A" HX vent valve 1E12-F074A	Torque switch set too low at 1	Valve torque out going to open position.	Adjusted torque switch to 2, within FLI requirements.
L55402	ARPM-"E" 1C51-K605GS	Excessive AC ripple on 5VDC Power Supply.	ARPM-"E" in bypass	Replaced capacitor C1 and bridge diode.
L55553	Pressure detector assembly 1PS-DG047	Bad mounting studs are either too short or stripped.	Need to remove pressure detector to fix.	Adjusted and replaced studs to appropriate length.
L55673	HPCS injection valve 1E22-F004	Internal binding within valve.	Valve binds up in intermediate position, tripping breaker.	Cleaned valve stem and lubricated.
L55707	H ₂ Recombiner cooling water valve	Internal binding within the valve.	Overloads trip while cycling valve.	Cleaned and lubricated stem.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L55783	"1A" DG Cooling water pump 1DG01P	Pump oil fill line broken	Pump inoperable	Fix line and refilled with oil.
L55849	Control Room HVAC Ammonia detector OXY-VC165A	Bad optic light.	Tripped during LIS-VC-053 and unable to reset.	Replaced Optic lights.
L55872	"1A" DG Differential relay 1487-DG002	Cut wire at connection	Differential relay inoperable.	Resoldered cut wire.
L55909	"1A" DG lube oil pump 1DG01K	Thermal overload on breaker keeps tripping.	Pump inoperable.	Replaced thermal overload.
L56076	Bus 143 Relaying 1AP07E	Loose connections and dirty slide contacts for bus pot compartment.	Relay Chatters	Cleaned and tightened appropriate contacts.
L56081	RHR Shutdown cooling temperature recorder 1812-R601	Bound up servo motor	Recorder was inoperable.	Replaced servo motor.
L56140	"1B" DG Fuel oil transfer pump 1D002P	Breaker position set too high.	Breaker trips on magnetics with switch in any position.	Adjusted to appropriate setpoint on breaker.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, logged during the reporting period, February 1 through February 28, 1986. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
86-001-00	01/10/86	"B" VC Ammonia Detector Trip, Due from loading gate misaligned or dirty photo cell optic.
86-002-00	01/20/86	SRV'S - S, E, U failed to meet lift tolerance, due to seat leakage.
86-003-00	01/25/86	Missed sample on Reactor Water pH, due to personnel error.
86-004-00	02/05/86	"A" VC Ammonia Detector trip, due to broken chemcassette tape.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORTDOCKET NO. 050-373UNIT LaSalle OneDATE 3/10/86COMPLETED BY James P. PetersTELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: November 1985 GROSS HOURS IN REPORTING PERIOD: 672
 2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
 4. REASONS FOR RESTRICTION (IF ANY): N/A
- | | THIS MONTH | YR TO DATE | CUMULATIVE |
|---|------------|------------|------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL | 0.0 | 0.0 | 12039.00 |
| 6. REACTOR RESERVE SHUTDOWN HOURS | 0.0 | 0.0 | 1642.00 |
| 7. HOURS GENERATOR ON LINE | 0.0 | 0.0 | 11642.00 |
| 8. UNIT RESERVE SHUTDOWN HOURS | 0.0 | 0.0 | 0.0 |
| 9. GROSS THERMAL ENERGY GENERATED (MWH) | 0.0 | 0.0 | 32213650 |
| 10. GROSS ELEC. ENERGY GENERATED (MWH) | 0.0 | 0.0 | 10499394 |
| 11. NET ELEC. ENERGY GENERATED (MWH) | -8847 | -14429 | 9990028 |
| 12. REACTOR SERVICE FACTOR | 0.0% | 0.0% | 63.4% |
| 13. REACTOR AVAILABILITY FACTOR | 0.0% | 0.0% | 72.1% |
| 14. UNIT SERVICE FACTOR | 0.0% | 0.0% | 61.3% |
| 15. UNIT AVAILABILITY FACTOR | 0.0% | 0.0% | 61.3% |
| 16. UNIT CAPACITY FACTOR (USING MDC) | -1.2% | -0.9% | 50.8% |
| 17. UNIT CAPACITY FACTOR (USING DESIGN MWe) | -1.2% | -0.9% | 48.8% |
| 18. UNIT FORCED OUTAGE RATE | 0.0% | 0.0% | 17.4% |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)
The First Refueling, Maintenance, Surveillance and Modification Outage began October 18, 1985 and will last 30 weeks.
 20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: May 3, 1986

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373

UNIT: LASALLE ONE

DATE: 3/10/86

COMPLETED BY: James P. Peters

TELEPHONE: (815) 357-6761

MONTH: FEBRUARY 1986

DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

1.	-13
2.	-13
3.	-13
4.	-13
5.	-13
6.	-13
7.	-13
8.	-13
9.	-13
10.	-13
11.	-14
12.	-14
13.	-13
14.	-13
15.	-13
16.	-14

DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

17.	-13
18.	-12
19.	-12
20.	-12
21.	-13
22.	-14
23.	-14
24.	-14
25.	-14
26.	-14
27.	-15
28.	-14
29.	N/A
30.	N/A
31.	N/A

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373UNIT NAME LaSalle OneDATE 3/10/86COMPLETED BY James P. PetersTELEPHONE (815)357-6761REPORT MONTH FEBRUARY 1986

NO.	DATE	TYPE	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				

There were no Unit shutdowns or power reductions during this reporting period due to the refuel outage in progress.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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There were no Safety Relief Valves Operated for Unit One during this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
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Reference Attached Pages 1-2.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-217-86	RHR Shutdown Cooling System	Install new Temperature Instrumentation.
1-218-86	1K31-F005C1	Repair Sealtite
1-220-86	1K51-F076	Repair Leaking Seat
1-221-86	1K12-F094	LES-EQ-112
1-228-86	"1A" DG Cooling Pump	Repair Pump Oiler
1-229-86	1K12-F074B	LES-EQ-112
1-230-86	1K12-F047B	Clean Worm Shaft Assembly
1-231-86	1K51-F045	LES-EQ-112
1-233-86	1K12-F016B	LES-EQ-112
1-240-86	HP-CY Spool Piece	Install Spool Piece
1-241-86	"A" and "B" WS Pumps	LES-EQ-112
1-242-86	1K22-C003	Lubrication
1-243-86	1K12-F018A	LES-EQ-112
1-246-86	1K22-F010	Isolate Line
1-251-86	1K12-F011B	Change Motor Grease
1-252-86	"1B" DG Motor AC	Troubleshoot and Repair
1-253-86	"1A" DG New Oil Pump	Test Breaker
1-254-86	1K12-F064A	Repair Valve
1-256-86	1K51-F010	LES-EQ-112
1-257-86	1K51-F019	LES-EQ-112
1-258-86	1K51-FC22	LES-EQ-112
1-259-86	1K51-F031	LES-EQ-112
1-260-86	1K51-F046	LES-EQ-112
1-261-86	1K51-F059	LES-EQ-112
1-265-86	"1A" DG Turbo's A/C Pump.	Repair/Replace Thermal Overload.
1-267-86	"B" RHR W.L. Pump Check Valve	Replace Check Valve Flange Bolts
1-268-86	1K12-F060B/75B	Repair Valves
1-274-86	1K12-F040A	Lubrication
1-277-86	1K12-F068A	Replace Sealtite.
1-280-86	RCIC	Lubrication Throttle Valve.
1-285-86	"B" RHR Sample Valve	Repair Valves
1-287-86	1K21-F001	LES-EQ-112
1-288-86	1DG035	LES-EQ-112
1-302-86	RCIC Throttle Valve	Assist. Mech. Maint.
1-311-86	"1A" DG Water Tight Door	Add Time Delay
1-319-86	1K12-F090B	Close Valve Till Testing Complete
1-333-86	1G33-F102	Regrease Limitorque
1-334-86	1K51-F063	Repack Valve
1-337-86	1K22-F012	Repair Motor
1-339-86	LPCS Spool Piece	Remove Spool
1-340-86	1K12-C300A	Lubrication

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-343-86	1E12-F049A	LRS-BQ-112
1-344-86	"1A" RHR Pump	Recouple
1-347-86	1E21-C001	Lubrication
1-349-86	1E21-F057	Close Closed Till System Restarted
1-352-86	1E12-F040A	Replace Snap Ring.
1-382-86	1E12-F041	Replace Packing
1-387-86	"1A" RHR Pump	Oil Sample
1-389-86	"1B" RHR WS Rad Monitor	Relocate Probe
1-405-86	DIV-I Water Leg Pump	Repair Pump
1-408-86	1E12-F092B	Keep Closed
1-409-86	1E12-F010A	Open for Draining
1-413-86	"1A" DG Maint. Switch	Cooling Water Pump Out of Service.
1-415-86	RHR System Pressure Switches	Install New Low Pressure Switches.

3. Off-Site Dose Calculation Manual

There were no changes to the ODCM during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no significant changes to the radioactive waste treatment system during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

FEBRUARY 1986

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System

I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by James P. Peters, telephone number (815)357-6761 extension 325.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

February 1-28

Feb. 1, 0001 Hours. The Unit entered February with the Reactor critical at 100% power and Generator on line at 1118 MWE.

Feb. 6, 1155 Hours. Reactor power reduced to 91% (1000 MWE) for Control Rod Manipulations.

Feb. 7, 0400 Hours. Reactor power restored to 100% (1117 MWE).

Feb. 8, 1210 Hours. Commenced power to drop to 64% (700 MWE) due to BPV #1 failing surveillance.

Feb. 8, 1445 Hours. BPV #1 repaired and began ramping up in power.

Feb. 9, 1500 Hours. Reactor power restored 99% (1100 MWE).

Feb. 10, 1500 Hours. Commenced power to drop of 250 MWE/Hr. due to BPV #1 failing surveillance.

Feb. 10, 2110 Hours. Generator taken off-line.

Feb. 11, 0125 Hours. Reactor shutdown (subcritical).

Feb. 11, 2050 Hours. Reactor went critical.

Feb. 12, 0300 Hours. Generator on-line and reactor ramping up in power.

Feb. 12, 2300 Hours. Reactor power at 72% (840 MWE).

Feb. 13, 0455 Hours. Reactor power reduced to 53% (596 MWE) due to Control Rod Manipulations.

Feb. 13, 2300 Hours. Reactor power restored to 91% (1005 MWE).

Feb. 14, 0200 Hours. Reactor power reduced to 65% (725 MWE) due to Control Rod Manipulations.

Feb. 16, 2300 Hours. Reactor power restored to 99% (1106 MWE).

II. MONTHLY REPORT FOR UNIT TWO (Continued)

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

February 1-28

Feb. 16, 1503 Hours. Reactor scram on load reject.

Feb. 17, 1906 Hours. Reactor went critical.

Feb. 18, 0355 Hours. Generator on-line.

Feb. 20, 0700 Hours. Reactor power restored to 84% (945 MWE) limited by rod pattern.

Feb. 20, 2100 Hours. Reactor power reduced to 51% (586 MWE) due to Control Rod Manipulations.

Feb. 23, 2000 Hours. Reactor power restored to 100% (1114 MWE).

Feb. 25, 0505 Hours. Reactor power decreased to 85% (966 MWE) due to Control Rod Manipulations and held for preconditioning ramps and LIS-NR-216.

Feb. 26, 1500 Hours. Reactor power restored to 100% (1116 MWE).

Feb. 28, 2400 Hours. Reactor power at 99% (1100 MWE).

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

There were no Amendments to facility license or Technical Specifications during this reporting period.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure change requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit Two during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L49202	"A" SBLC Pump 2C41-C001A	Bad packing and Piston Shaft.	Water leaks about 20 Gal/Min into drain barrell.	Installed new packing and piston shafts.
L49982	HPCS Pump Suction Pressure Indicating Switch 2E22-N015.	Incorrect valves on Data Sheet.	Reading 25 psig with HPCS System off, when should read 10 psig.	Recalibrated pressure switch.
L54030	RCIC Pump Suction from CY 2E51-F010	Need torque switch	Improper valve indication	Installed limiter plate.
L54832	HCU-22-27 2C11-D001-111	Bad cartridge intervals	Valve leaks by when opening and closing.	Repaired cartridge valve.
L54833	HCU-14-39 2C11-D001-111	Bad cartridge intervals	Valve leaks by when opening and closing.	Repaired cartridge valve.
L54834	HCU-14-27 2C11-D001-111	Bad cartridge intervals	Valve leaks by when opening and closing.	Repaired cartridge valve.
L54837	HCU-54-35 2C11-D001-111	Bad cartridge intervals	Valve leaks by when opening and closing.	Repaired cartridge valve.
L54838	HCU-42-15 2C11-D001-111	Bad cartridge intervals	Valve leaks by when opening and closing.	Repaired cartridge valve.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L55510	H ₂ Recombiner Cross-tie valve 2HG009	Blown fuse resulting from pinched wire causing a short.	Valves has no light indication	Replaced wire and lugs.
L55627	Main Steam Line valve 2B21-F067C	Bad torque switch	Seal-in the close direction does not work.	Installed new torque switch.
L55629	#1 Bypass valve 2B21-MSBPV-1	Loose packing bolts.	Valve packing leaks.	Tightened packing bolts.
L55677	APRM-A 2C51-K6056M	Bad meter	Panel indication sticks at 65%	Replaced meter and calibrated.
L55708	"C" RHR Pump Min. Flow valve 2E12-F064C	2E12-N010CA Setpoint was too low.	Valve does not open when should	Recalibrated to appropriate setpoint.
L55871	RCIC Outboard Steam Isolation 2E51-F008	Improper packing	Steam leak around stem.	Added 1 ring of packing and adjusted till leak stopped.
L55890	RCIC Outboard Steam Isolation 2E51-F008	Unidentified Jumpers and loose screw on limits	Valve failed to close intermittently.	Replaced unidentifiable jumpers, tighten loose screw and limits.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L55910	ADS System "B" 2B21-K35B	Bad Agastat GPI Time Delay Relay.	During LES-NB-102, setpoint was determined to be inconsistent.	Calibrated new agastat to 101 seconds.
L56109	MSIV-LCS temperature indicator 2E32-R602J	Bad meter	Failed down scale, 30 day time clock.	Temporary system change 1-345-86 to install meter from Unit #1.
L56143	RCIC Outboard Steam Line Isolation Valve 2E51-F008.	Defective contacts on Control switch.	Valve failed to close from control switch.	Replaced contact blocks on switch.
L56234	MSIV-LCS Heater Temperature indicator (N) TY-2E32-R601N	Temperature indicator out of tolerance	Improper temperature indication.	Recalibrated temperature indicator.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, logged during the reporting period, February 1 through February 28, 1986. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
86-001-00	01/14/86	Failure of ACB 2413 to close, due to faulty relay contacts.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374UNIT LaSalle TwoDATE 3/10/86COMPLETED BY James P. PetersTELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: February, 1986 GROSS HOURS IN REPORTING PERIOD: 672
 2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
 4. REASONS FOR RESTRICTION (IF ANY): N/A
- | | THIS MONTH | YR TO DATE | CUMULATIVE |
|---|----------------|----------------|-----------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL | <u>624.54</u> | <u>1368.54</u> | <u>6757.94</u> |
| 6. REACTOR RESERVE SHUTDOWN HOURS | <u>29.83</u> | <u>29.83</u> | <u>1716.93</u> |
| 7. HOURS GENERATOR ON LINE | <u>605.3</u> | <u>1349.3</u> | <u>6585.6</u> |
| 8. UNIT RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>0.0</u> | <u>0.0</u> |
| 9. GROSS THERMAL ENERGY GENERATED (MWH) | <u>1798080</u> | <u>3964104</u> | <u>19472656</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH) | <u>601090</u> | <u>1325871</u> | <u>6435940</u> |
| 11. NET ELEC. ENERGY GENERATED (MWH) | <u>582879</u> | <u>1289889</u> | <u>6113104</u> |
| 12. REACTOR SERVICE FACTOR | <u>92.9%</u> | <u>96.6%</u> | <u>56.4%</u> |
| 13. REACTOR AVAILABILITY FACTOR | <u>97.4%</u> | <u>98.8%</u> | <u>70.8%</u> |
| 14. UNIT SERVICE FACTOR | <u>90.1%</u> | <u>95.3%</u> | <u>54.9%</u> |
| 15. UNIT AVAILABILITY FACTOR | <u>90.1%</u> | <u>95.3%</u> | <u>54.9%</u> |
| 16. UNIT CAPACITY FACTOR (USING MDC) | <u>83.7%</u> | <u>87.9%</u> | <u>31.1%</u> |
| 17. UNIT CAPACITY FACTOR (USING DESIGN MWe) | <u>80.5%</u> | <u>84.5%</u> | <u>29.8%</u> |
| 18. UNIT FORCED OUTAGE RATE | <u>9.9%</u> | <u>4.7%</u> | <u>23.2%</u> |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
There are no shutdowns scheduled over the next 6 months.
 20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP N/A

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374

UNIT: LASALLE TWO

DATE: 3/10/86

COMPLETED BY: James P. Peters

TELEPHONE: (315) 357-6761

MONTH: February 1986

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	1073
2.	1078
3.	1074
4.	1072
5.	1067
6.	1032
7.	1076
8.	986
9.	1047
10.	763
11.	-13
12.	503
13.	853
14.	918
15.	1040
16.	667

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	-12
18.	408
19.	835
20.	872
21.	757
22.	916
23.	1030
24.	1055
25.	981
26.	1070
27.	1075
28.	1069
29.	N/A
30.	N/A
31.	N/A

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373

UNIT NAME LaSalle One

DATE 3/10/86

COMPLETED BY James P. Peters

TELEPHONE (815)357-6761

REPORT MONTH FEBRUARY 1986

NO.	DATE	TYPE	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				
15	2/10/86	F	29.83	A	5	Failure of #1 Bypass Valve.
16	2/16/86	F	36.87	A	3	Reactor scram on load reject due to a phase to phase fault on a transmission line. The A phase OCB 1-9 failed to trip (out of tolerance on trip coils) resulting in the 2 East Main Transformer.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATIONS</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
02/16/86	2B21-F013U 2B21-F013K 2B21-F013D 2B21-F013S 2B21-F013E 2B21-F013C	1, Automatic 1, Automatic 1, Automatic 1, Automatic 1, Automatic 1, Automatic	98% power	The unit scramed on load reject, and SRV's 2B21-F013U,K,D,S,E and C. All immediately opened to relieve reactor pressure. These actuations were automatic due to reactor over pressurization. The valves were open for U(52 sec.), K(15 sec.), D(12 sec.), S(15 sec.), E(12 sec.), and C(12 sec.) respectively. The suppression pool heated up 7.1°F as a result of these actuations. All SRV's which opened appeared to properly reseal by tailpipe temperature indications.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-59-86	2E51-F008	Inspect Wiring
2-64-86	2E51-F008	Isolate RCIC System
2-72-86	RCIC Vacuum Pump	Troubleshoot.
2-74-86	"2A" DG Air Compressor	Replace second stage discharge Cap Gasket.
2-76-86	2E51-F008	Wiring Inspection.

3. Off-Site Dose Calculation Manual

There were no changes to the ODCM during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the radioactive waste treatment system during this reporting period.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

March 10, 1986

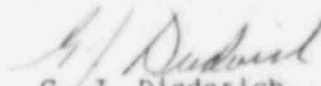
Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station for the period February 1, through
February 28, 1986.

Very truly yours,


G. J. Diederich
Station Manager
LaSalle County Station

GJD/RJR/crh

Enclosure

xc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
D. P. Galle, CECO
D. L. Farrar, CECO
INPO Records Center
L. J. Anastasia, PIP Coordinator SNED
J. E. Ellis, GE Resident
H. E. Bliss, Nuclear Fuel Services Manager
C. F. Dillon, Senior Financial Coordinator, LaSalle

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