

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) EDWIN I. HATCH, UNIT II										DOCKET NUMBER (2) 0 5 0 0 0 13 16 6				PAGE (3) 1 OF 0 13	
TITLE (4) ENGINEERED SAFETY FEATURE ACTUATION (RWCU ISOLATION)															
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)		
0 7	2 6	8 5	8 5	0 2	0	0 7	2 6	8 5					0 5 0 0 0		
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5 (Check one or more of the following) (11)															
OPERATING MODE (9)		1		20.402(b)		20.406(c)		<input checked="" type="checkbox"/>		50.73(a)(2)(iv)		73.71(b)			
POWER LEVEL (10)		1 0 0		20.406(a)(1)(i)		50.36(a)(1)				50.73(a)(2)(v)		73.71(c)			
				20.406(a)(1)(ii)		50.36(c)(2)				50.73(a)(2)(vi)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
				20.406(a)(1)(iii)		50.73(a)(2)(i)				50.73(a)(2)(vii)(A)					
				20.406(a)(1)(iv)		50.73(a)(2)(ii)				50.73(a)(2)(vii)(B)					
				20.406(a)(1)(v)		50.73(a)(2)(iii)				50.73(a)(2)(ix)					
LICENSEE CONTACT FOR THIS LER (12)															
NAME Raymond D. Baker, Nuclear Licensing Manager - Hatch										TELEPHONE NUMBER AREA CODE 4 0 4 5 2 6 1 7 0 1 6					
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)															
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS						
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR	
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO					
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)															
<p>At approximately 1215 CDT on 07/26/85, with the unit operating at 2436 MWt (approximately 100% power) following performance of plant procedure HNP-2-1326, plant personnel were placing the reactor water cleanup (RWCU) system in service per plant procedure HNP-2-1325. At that time, they noted that the RWCU pump inboard (2G31-F001) and outboard (2G31-F004) suction isolation valves had isolated on high system differential flow.</p> <p>After an initial investigation, plant personnel determined that the RWCU system isolation apparently occurred because filter demineralizer vessel 2G31-D002B was not completely filled with water when the RWCU system was placed in service. Consequently, the RWCU system experienced a water surge and isolated on high system differential flow.</p> <p>After completing the investigation, plant personnel satisfactorily placed the RWCU system in service per HNP-2-1325 on 07/26/85 at approximately 1240 CDT.</p> <p>During further engineering evaluation, the actual root cause of the high differential flow reading which resulted in the RWCU isolation was determined to be leakage past the RWCU loop "A" resin strainer drain valves (2G31-F126A and 2G31-F127A). Investigation and planning for corrective actions to prevent leakage through those valves in the future is underway.</p> <p>This event resulted in no adverse plant safety consequences and did not affect the health and safety of the public.</p>															
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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  EDWIN I. HATCH, UNIT II	DOCKET NUMBER (2)  0500036685-020102 OF 03	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This 30 day LER is reportable per 10CFR 50.73(a)(2)(iv) because this event was the unplanned actuation of an engineered safety feature (primary containment isolation valves 2G31-F001 and 2G31-F004).

At approximately 1215 CDT on 07/26/85, with the unit operating at 2436 MWt (approximately 100% power) following performance of the "REACTOR WATER CLEANUP DEMINERALIZER" procedure (HNP-2-1326), plant personnel were placing the reactor water cleanup (RWCU) system in service per the "REACTOR WATER CLEANUP SYSTEM" procedure (HNP-2-1325). At that time, they noted that the RWCU pump inboard (2G31-F001) and outboard (2G31-F004) suction isolation valves had isolated on high system differential flow.

After an initial investigation, plant personnel identified the most likely cause to be that the filter demineralizer vessel 2G31-D002B had apparently not remained filled after the backwash and precoat cycle per procedure HNP-2-1326. Consequently, when RWCU system "2B" was placed in service per procedure HNP-2-1325, RWCU water surged into the incompletely filled filter demineralizer vessel (2G31-D002B) causing RWCU pump suction isolation valves 2G31-F001 and 2G31-F004 to isolate on high RWCU system differential flow.

After completing the initial investigation, plant personnel satisfactorily placed the RWCU system in service per HNP-2-1325 on 07/26/85 at approximately 1240 CDT.

Subsequent to the initial investigation an engineering evaluation was performed which determined the actual cause of this occurrence (and past similar events) to be that the RWCU loop "A" resin strainer (2G31-D004A) drain valves (2G31-F126A and 2G31-F127A) were allowing water to flow into radwaste cleanup phase separator tank 2G11-D005A when RWCU loop "A" was placed in service. The flow resulting from that drain valve leakage caused the high RWCU system differential flow signal and subsequent isolation of valves 2G31-F001 and 2G31-F004. The root cause of that drain valve leakage was found to be as follows:

- a. The adapter plate (also used as a local position indicator) on drain valve 2G31-F127A was incorrectly slotted by the valve manufacturer (i.e., the slot was manufactured 90 degrees from its proper orientation). Consequently, the remote position indicator showed the valve closed when it was actually in the open position.
- b. Drain valve 2G31-F126A did not operate properly on a signal from the associated remote operator because of excessive friction between the operator plate and the adapter plate on that valve. The excessive friction caused the valve stem to begin travel before the adapter plate contacted the lugs on the operator plate. Consequently, the valve stem overtraveled and resulted in an improperly positioned valve ball which permitted water to flow through that valve while it was supposedly closed.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Subsequent to the engineering evaluation of this event, the system engineer issued a memo to the operations department which advises verification of the valve position by physically checking the valve local position indicators. Additionally, a new correctly slotted valve adapter plate will be installed on resin strainer drain valve 2G31-F127A when it is received from the vendor. An investigation into long term corrective action for valve 2G31-F126A is in progress.

This event had no actual or potential adverse safety consequences (i.e., the isolation of valves 2G31-F001 and 2G31-F004 had no impact on safe operation of the plant); nor were other systems in Unit 1 or Unit 2 affected. This event did not affect the health and safety of the public.

Past corrective action did not prevent this event because the above noted condition of valves 2G31-F001 and 2G31-F004 was not known prior to the engineering evaluation which was initiated as a result of this occurrence.

Similar past events where the RWC system has isolated on high system differential flow are described in the following LERs: 50-321/1984-010 on 08/20/84; 50-366/1984-007 on 09/08/84; 50-366/1984-031 on 10/31/84; 50-321/1985-019 on 04/30/85; 50-366/1985-021 on 07/07/85.

Georgia Power Company  
333 Piedmont Avenue  
Atlanta, Georgia 30308  
Telephone 404 526-6526

Mailing Address:  
Post Office Box 4545  
Atlanta, Georgia 30302

L. T. Gucwa  
Manager Nuclear Safety and  
Licensing Department



SL-412  
0166C

February 28, 1986

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

Attached is Licensee Event Report 50-366/1985-020, Rev. 1. This report meets the reporting requirements of 10 CFR 50.73(a)(2)(iv).

Very truly yours,

L. T. Gucwa

CBS/lc

Attachment

c: Mr. J. T. Beckham, Jr.  
Mr. H. C. Nix, Jr.  
NRC-Region II  
GO-NORMS

IE22  
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