

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) H. B. Robinson, Unit No. 2										DOCKET NUMBER (2) 0 5 0 0 0 2 6 1										PAGE (3) 1 OF 0 2																			
TITLE (4) Steam Generator Snubbers																																							
EVENT DATE (5) MONTH DAY YEAR 0 1 3 1 8 6 8 6										LER NUMBER (6) YEAR SEQUENTIAL NUMBER REVISION NUMBER — 0 0 7 — 0 0										REPORT DATE (7) MONTH DAY YEAR 0 3 0 6 8 6										OTHER FACILITIES INVOLVED (8) FACILITY NAMES DOCKET NUMBER(S) 0 5 0 0 0 0									
OPERATING MODE (9) POWER LEVEL (10) 0										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11) 20.402(a) 20.406(a) 50.73(a)(2)(iv) 73.71(b) 20.406(a)(1)(i) 50.36(a)(1) 50.73(a)(2)(iv) 73.71(c) 20.406(a)(1)(ii) 50.36(a)(2) X 50.73(a)(2)(iv) OTHER (Specify in Abstract below and in Text, NRC Form 306A) 20.406(a)(1)(iii) 50.73(a)(2)(i) 50.73(a)(2)(viii)(A) 20.406(a)(1)(iv) 50.73(a)(2)(ii) 50.73(a)(2)(viii)(B) 20.406(a)(1)(v) 50.73(a)(2)(iii) 50.73(a)(2)(ix)																													
LICENSEE CONTACT FOR THIS LER (12) NAME G. Honma TELEPHONE NUMBER AREA CODE 8 0 3 3 8 3 - 4 5 2 4																																							
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																							
CAUSE	SYSTEM	COMPONENT	MANUFAC TURE	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFAC TURE	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFAC TURE	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFAC TURE	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFAC TURE	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFAC TURE	REPORTABLE TO NPROS										
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SUPPLEMENTAL REPORT EXPECTED (14) YES (If yes, complete EXPECTED SUBMISSION DATE) X NO																																							
EXPECTED SUBMISSION DATE (15) MONTH DAY YEAR																																							

ABSTRACT (Limit to 1400 spaces; i.e., approximately fifteen single-space typewritten lines) (16)

Abstract

On February 4, 1986, at approximately 2100 hours, with the Plant in a refueling outage, it was determined that the snubber banks for Steam Generators (S/G) "A" and "B" would be declared inoperable in accordance with Technical Specification (T. S.) 4.13.1.d. During preparation for the refueling interval surveillance test (EST-032), the common hydraulic fluid reservoir and associated tubing for each bank of four (4) snubbers were observed to be empty. T. S. 4.13.1.d states "... when the fluid port of a hydraulic snubber is found to be uncovered, the snubber shall be determined inoperable..."

Inspections revealed the leak to be from a fitting between a check valve on the supply tubing and the control valve. Apparently, the fittings were not properly tightened following the previous maintenance surveillance testing which occurred during the Steam Generator Replacement Outage in 1984.

Following scheduled testing and tubing reinstallation, the hydraulic fluid system will be pressure tested to verify fitting integrity. Additionally, a periodic check of hydraulic fluid level in the common reservoir (one reservoir for each bank of four (4) snubbers) will be conducted during operation.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
H. B. Robinson, Unit No. 2	0500026186	007	00	02	OF	02

TEXT (If more space is required, use additional NRC Form 365A's) (17)

Event Description

On February 4, 1986, at approximately 2100 hours, with the Plant in a refueling outage, it was determined that the snubber banks for Steam Generators (S/G) "A" and "B" would be declared inoperable in accordance with Technical Specification (T. S.) 4.13.1.d. This specification states "... when the fluid port of a hydraulic snubber is found to be uncovered, the snubber shall be determined inoperable..."

On January 31, 1986, Plant cooldown was in progress in preparation for a refueling outage. While preparing to perform the refueling interval surveillance test (EST-032) on the Steam Generators (S/G) snubbers, the reservoirs for "A" and "B" S/G snubber banks were observed to be empty. A plan was formulated to identify the exact location of the leak and to see if fluid was present in the control valve and snubber cylinder. This would determine snubber operability in accordance with T. S. 4.13.1.d.

On February 4, 1986, the supply side tubing fitting to the control valve was disconnected to determine fluid availability to the control valve. Observations showed that the supply side of the tubing was dry to the control valve on both "A" and "B" S/G snubber banks. Although there was no fluid observed on the reservoir (supply) side, fluid was observed to be available in each of the snubber cylinders. A pressure test was then performed on the "A" and "B" S/G snubber hydraulic systems to determine the location of the leak. A leak, in both cases, was observed on the fitting between the check valve on the supply tubing and the control valve.

Although fluid was available in each of the snubber cylinders on both banks, the tubing to each control valve (one per bank) was dry causing the control valve ports to be uncovered. Therefore, the "A" and "B" S/G snubber banks would have been declared inoperable.

Cause/Corrective Action

All S/G snubbers (12) were removed during the Steam Generator Replacement Outage in 1984. Apparently, during reinstallation, the fittings were improperly tightened.

Following scheduled testing and tubing reinstallation this outage, the hydraulic fluid systems on all three S/G snubber banks will be pressure tested to ensure fitting integrity. Additionally, a periodic check of the hydraulic fluid level in the common reservoir (one reservoir for each bank of four (4) snubbers) will be conducted during operation.

* Baxter Fluid Power (Anker-Holth)



Carolina Power & Light Company

ROBINSON NUCLEAR PROJECT DEPARTMENT
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HARTSVILLE, SOUTH CAROLINA 29550
MAR 06 1986

Robinson File No: 13510C

Serial: RNP/86-888

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261
LICENSE NO. DPR-23
LICENSEE EVENT REPORT 86-007

Dear Sir:

In accordance with 10CFR50.73, Licensee Event Report System, the enclosed Licensee Event Report is submitted. This report fulfills the requirements for a written report within (30) days of a reportable event and is in accordance with the format set forth in NUREG-1022, September, 1983.

Very truly yours,

R. E. Morgan
General Manager
H. B. Robinson S. E. Plant

CH/JCS:sdm

Enclosure

cc: INPO
H. E. P. Krug
J. N. Grace

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