

DOCKET NO.: 50-354
UNIT: Hope Creek
DATE: 04/08/97
COMPLETED BY: R. Phillips
TELEPHONE: (609) 339-2735

OPERATING DATA REPORT

OPERATING STATUS

1. Reporting Period March 1997 Gross Hours in Report Period 744
2. Currently Authorized Power Level (MWt) 3293
Max. Depend. Capacity (MWe-Net) 1031
Design Electrical Rating (MWe-Net) 1067
3. Power Level to which restricted (if any) (MWe-Net) None
4. Reasons for restriction (if any)

	<u>This</u> <u>Month</u>	<u>Yr To</u> <u>Date</u>	<u>Cumulativ</u> <u>e</u>
5. No. of hours reactor was critical	<u>744.0</u>	<u>2160</u>	<u>75883.1</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
7. Hours generator on line	<u>744</u>	<u>2160</u>	<u>74720</u>
8. Unit reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
9. Gross thermal energy generated (MWH)	<u>2411356</u>	<u>7004237</u>	<u>238872484</u>
10. Gross electrical energy generated (MWH)	<u>818720</u>	<u>2379970</u>	<u>79274183</u>
11. Net electrical energy generated (MWH)	<u>787666</u>	<u>2289595</u>	<u>75761314</u>
12. Reactor service factor	<u>100.0</u>	<u>100.0</u>	<u>84.2</u>
13. Reactor availability factor	<u>100.0</u>	<u>100.0</u>	<u>84.2</u>
14. Unit service factor	<u>100.0</u>	<u>100.0</u>	<u>82.9</u>
15. Unit availability factor	<u>100.0</u>	<u>100.0</u>	<u>82.9</u>
16. Unit capacity factor (using MDC)	<u>102.7</u>	<u>102.8</u>	<u>81.5</u>
17. Unit capacity factor (using Design MWe)	<u>99.2</u>	<u>99.3</u>	<u>78.8</u>
18. Unit forced outage rate	<u>0.0</u>	<u>0.0</u>	<u>4.5</u>
19. Shutdowns scheduled over next 6 months (type, date, & duration): Refueling Outage, September 6, 1997, 60 days			
20. If shutdown at end of report period, estimated date of start-up:			

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OPERATING DATA REPORT
UNIT SHUTDOWNS AND POWER REDUCTIONS

MONTH MARCH 1997

NO.	DATE	TYPE	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTION/COMMENTS
		F=FORCED S=SCHEDULED				
n/a						

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AVERAGE DAILY UNIT POWER LEVEL

MONTH MARCH 1997

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>995</u>	17	<u>1066</u>
2	<u>1049</u>	18	<u>1062</u>
3	<u>1067</u>	19	<u>1065</u>
4	<u>1061</u>	20	<u>1061</u>
5	<u>1067</u>	21	<u>1059</u>
6	<u>1057</u>	22	<u>1051</u>
7	<u>1064</u>	23	<u>1094</u>
8	<u>1058</u>	24	<u>1039</u>
9	<u>1068</u>	25	<u>1078</u>
10	<u>1068</u>	26	<u>1051</u>
11	<u>1052</u>	27	<u>1055</u>
12	<u>1063</u>	28	<u>1051</u>
13	<u>1067</u>	29	<u>1053</u>
14	<u>1025</u>	30	<u>1020</u>
15	<u>1099</u>	31	<u>1063</u>
16	<u>1065</u>		

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REFUELING INFORMATION

MONTH MARCH 1997

1. Refueling information has changed from last month:

Yes No X

2. Scheduled date for next refueling (RF07): 9/6/97

3. Scheduled date for restart following refueling:
11/5/97

4A. Will Technical Specification changes or other license
amendments be required?

Yes No X

B. Has the Safety Evaluation covering the COLR been reviewed by
the Station Operating Review Committee (SORC)?

Yes No X

If no, when is it scheduled? To Be Determined for Cycle 8
COLR

5. Scheduled date(s) for submitting proposed licensing action:

Not required.

6. Important licensing considerations associated with refueling:

N/A

7. Number of Fuel Assemblies:

A. Incore 764
B. In Spent Fuel Storage 1472

8. Present licensed spent fuel storage capacity: 4006
Future spent fuel storage capacity: 4006

9. Date of last refueling that can be discharged 5/3/2006
to spent fuel pool assuming the present licensed capacity:
(EOC13)

(Does allow for full-core off-load)
(Assumes 244 bundle reloads every 18 months until then)
(Does not allow for smaller reloads due to improved fuel)

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MONTHLY OPERATING SUMMARY

MONTH MARCH 1997

- The Hope Creek Generating Station remained on-line for the entire month and operated at 100% power for the month of March 1997. There were two load reductions which are identified below.
- Power was reduced to 87% on March 1, 1997, starting at 0105 hours to perform monthly turbine valve testing. The unit was returned to 100% power on March 1, 1997, at 1325 hours.
- Power was reduced to 87% on March 30, 1997, starting at 0209 hours to perform turbine valve testing and rod shuffle. The unit was returned to 100% power on March 30, 1997, at 1045 hours.
- At the end of the month the unit had been on-line for 144 days.

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SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE HOPE CREEK GENERATING STATION

MONTH MARCH 1997

The following items completed during February 1997 have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes Summary of Safety Evaluations

Replacement of Air-Operated Valves in the Safety Auxiliary Cooling System (SACS), 4EC-03612 Pkgs. 2, 4, 6, and 7. The Standby Diesel Generator (SDG) room cooler water supply valves in the SACS were replaced. The previous valves were carbon steel air actuated flexible-wedge gate valves. The replacement valves are stainless steel air actuated ball valves which are more suitable to the application. UFSAR Figure 9.2-5 was changed to show the type and material of the replacement valves and actuators. UFSAR Table 3.9-18 was changed to show the valve/actuator replacement type. This replacement does not change the function of the valves or the function of the affected systems. The change does not alter the ability of the SACS, the SDG Room Recirculation System, or the SDGs to perform their intended safety function. The affected systems will function in accordance with the original design and licensing basis of the plant.

Therefore, this design change does not increase the probability or consequences of an accident previously described in the UFSAR and does not involve an Unreviewed Safety Question.

Replacement of Air-Operated Valves in the Safety Auxiliaries Cooling System (SACS), 4EC-03612, Pkg. 9. The Residual Heat Removal (RHR) pump room cooler water supply valves in the SACS were replaced. The previous valves were carbon steel air actuated flexible-wedge gate valves. The replacement valves are stainless steel air actuated ball valves which are more suitable to the application. UFSAR Figure 9.2-4 was changed to show the type and material of the replacement valves/actuators. UFSAR Table 3.9-18 was changed to show the valve/actuator replacement type. This replacement does not change the function of the valves or the function of the affected systems. The change does not alter the ability of the SACS, the Equipment Area Cooling System (EACS), or the RHR pump room unit coolers to perform their intended safety functions. The affected systems will function in accordance with the original design and licensing basis of the plant.

Therefore, this design change does not increase the probability or consequences of an accident previously described in the UFSAR and does not involve an Unreviewed Safety Question.

Replacement of Air-Operated Valves in the Safety Auxiliaries Cooling System (SACS), 4EC-03612, Pkg. 27. The Core Spray (CS) pump room cooler water supply valves in the SACS were replaced. The previous valves were carbon steel air actuated flexible-wedge gate valves. The replacement valves are stainless steel air actuated ball valves which are more suitable to the application. UFSAR Figure 9.2-4 was changed to show the type and material of the replacement valves/actuators. UFSAR Table 3.9-18 was changed to show the valve/actuator replacement type. This replacement does not change the function of the valves or the function of the affected systems. The change does not alter the ability of the SACS, the Equipment Area Cooling System (EACS), or the CS pump room unit coolers to perform their intended safety functions. The affected systems will function in accordance with the original design and licensing basis of the plant.

Therefore, this design change does not increase the probability or consequences of an accident previously described in the UFSAR and does not involve an Unreviewed Safety Question.

Remove Flow Switch 1EPFS-2225B, 4HE-00356, Pkg. 2.. This design change removed the service water screen and backwash

flow switch, 1EPFS-2225B, and installed gaskets and a blind flange in place of the flow switch. Stress and seismic evaluations have been performed to demonstrate the adequacy of the change. The replacement of the flow switch with a blind flange does not affect the amount of flow being provided to the Reactor Auxiliary Cooling System (RACS) and Safety Auxiliary Cooling System (SACS) heat exchangers during plant conditions, transients, and accident conditions. The change does not: 1) change, degrade, or prevent actions described or assumed in any accident described in the SAR, 2) alter any assumptions previously made in evaluating radiological consequences of any accident described in the SAR, 3) affect the mitigation of the radiological consequences of any accident described in the SAR, 4) affect a fission product barrier, or 5) change the composition or inventory of radioactivity releases. This change does not adversely affect the operability of the Station Service Water System (SSWS).

Therefore, this design change does not increase the probability or consequences of an accident previously described in the UFSAR and does not involve an Unreviewed Safety Question.

Procedures Summary of Safety Evaluations

THC.CH-SA.RC-0004(Q), Temporary Balance of Plant (BOP) Sampling Program. This temporary BOP Sampling Program establishes chemistry parameters, specifications, sampling frequencies, and sampling locations to meet Updated Final Safety Analysis Report (UFSAR) sampling requirements during the BOP sample station replacement. The proposed procedure does not have any negative effects on safety related functions and does not compromise any safety related system or components or prevent safe shutdown of the plant.

Therefore, this temporary procedure does not increase the probability or consequences of an accident previously described in the UFSAR and does not involve an Unreviewed Safety Question.

Other Summary of Safety Evaluation

Safety Evaluation H97-006, On-Site Transportation of Salem Unit 1's Original Steam Generators (OSGs) including Temporary Parking of the OSGs. A temporary parking area, for storage of the OSGs prior to their shipment offsite, was erected near the low level waste storage facility, by creating a Radiological Controlled Area (RCA) and erecting temporary concrete block shielding. This safety evaluation was prepared to evaluate the effect of the RCA on the Hope

Creek Generating Station. The safety evaluation demonstrated that neither the RCA, nor the transportation, nor the temporary parking of the OSGs adversely affected any structure, system or component. The temporary parking area, as erected and controlled, does not present any radiological safety concerns.

Therefore, implementation of this change does not increase the probability or consequences of an accident previously described in the UFSAR and does not involve an Unreviewed Safety Question.

UFSAR Change Notices Summary of Safety Evaluations

Temporary Modifications Summary of Safety Evaluations

Deficiency Reports Summary of Safety Evaluations

There were no changes in these categories implemented during February 1997.