NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
PALISADES PLANT	0 5 0 0 0 2 5 5	1 OF 0 2

TITLE (4)

Primary Coolant System Unidentified Leakage Greater Than Limit

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
1 1	1 9	8 4	8 4	0 2 5	0 1	1 2	0 6	8 5	NA	0 5 0 0 0
									NA	0 5 0 0 0

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 1 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	X OTHER (Specify in Abstract below and in Text, NRC Form 366A) Voluntary
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Robert A Fenech; Technical Engineer; Palisades	AREA CODE 6 1 6 7 6 4 - 8 9 1 3

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	C B	R V	F 0 3 5	Y							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On November 19, 1984, with the Plant in hot shutdown, a leak rate test indicated primary coolant system unidentified leakage to be in excess of Technical Specification limits. Investigation determined that the leakage was from a relief valve in the Chemical and Volume Control System. The valve was exercised and a subsequent leak rate test indicated normal leak rates within Technical Specification limits.

This event is not reportable per 10CFR50.73 but this report is submitted as a voluntary report.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

On November 19, 1984, with the Plant in hot shutdown, a two hour duration primary coolant system [AB] leak rate test indicated that unidentified leakage was in excess of the Technical Specification 3.1.5(a) limit. Calculations showed that unidentified leakage was 4.22 gallons per minute. The Technical Specifications limit for unidentified leakage is one gallon per minute.

Subsequent investigation determined that relief valve RV-2082 [RV;CB] was the source of the leakage. The valve was manually lifted, and a follow-up four hour duration leak test indicated that RV-2082 had seated properly. Calculations showed that unidentified leakage was reduced to 0.796 gallons per minute. This leak rate is within Technical Specifications limits.

Relief valve RV-2082 provides overpressure protection for the primary coolant pump [P;AB] controlled bleed-off piping [CB]. Evaluation of relief valve RV-2082 revealed that a bent stem had caused the valve to bind in an open position. No cause has been determined for the bent stem. The stem problem is not considered to have generic implications. Relief valve RV-2082 will be repaired when maintenance preparations are completed.

Although the leak rate exceeded Technical Specifications limits, no threat to public health or safety resulted. The source was easily identified, and the leakage was minimal. No reduction in plant operating conditions was required.

This event was determined not to be reportable per 10CFR50.73 since the event did not result in a condition prohibited by Technical Specifications and appropriate corrective action was taken in accordance with the applicable Technical Specifications action statement.

Consumers Power Company

General Offices: 1945 West Parnall Road, Jackson, MI 49201 • (517) 788-0550

December 6, 1985

US Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

DOCKET 50-255 - LICENSE DPR-20 - PALISADES PLANT -
LICENSEE EVENT REPORT 84-025 - REVISION 1 - PRIMARY COOLANT SYSTEM
UNIDENTIFIED LEAKAGE GREATER THAN LIMIT

Licensee Event Report (LER) 84-025, "Primary Coolant System Unidentified Leakage Greater Than Limit" dated December 19, 1984, was submitted to the NRC as a reportable event per the requirements of 10CFR50.73(a)(2)(i).

Consumers Power Company has reviewed this report and determined that the event did not result in a condition prohibited by Technical Specifications. Appropriate corrective action was taken in accordance with the applicable Technical Specification action statement. Therefore, no reporting requirement specified by 10CFR50.73 would apply to this event. Revision 1 to Licensee Event Report 84-025 changes the reportability requirement to "other - voluntary."

Brian D Johnson

Brian D Johnson
Staff Licensing Engineer

CC Administrator, Region III, USNRC
NRC Resident Inspector - Palisades

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OC1285-0002-NL02